

original proposed no significant hazards consideration determination.

The Commission's related evaluation of the amendment is contained in the Safety Evaluation dated July 26, 2018.

No significant hazards consideration comments received: No.

Southern Nuclear Operating Company, Docket Nos. 52-025 and 52-026, Vogtle Electric Generating Plant (VEGP), Units 3 and 4, Burke County, Georgia

Date of amendment request: June 15, 2018 and supplemented on June 25, 2018 and July 10, 2018.

Description of amendment: The amendment revises commitments related to the construction fitness-for-duty (FFD) program described in the VEGP Units 3 and 4 Updated Final Safety Analysis Report. Specifically, the change involves the creation of a new type of FFD Authorization that allows construction workers temporary access to the construction site pending completion of all pre-access FFD requirements. The individuals will not be given assignments to work on safety or security-related structures, systems, and components prior to the completion of the FFD requirements.

Date of issuance: August 29, 2018.

Effective date: As of the date of issuance and shall be implemented within 30 days of issuance.

Amendment Nos.: 141 (Unit 3) and 140 (Unit 4). A publicly-available version is in ADAMS under Accession No. ML18214A659; documents related to this amendment are listed in the Safety Evaluation enclosed with the amendment.

Facility Combined Licenses No. NPF-91 and NPF-92: Amendment revised the Facility Combined Licenses.

Date of initial notice in Federal Register: June 27, 2018 (83 FR 30199). The supplements dated June 25, 2018, and July 10, 2018, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the NRC staff's original proposed no significant hazards consideration determination as published in the **Federal Register**.

The Commission's related evaluation of the amendment is contained in the Safety Evaluation dated August 29, 2018.

No significant hazards consideration comments received: No.

Tennessee Valley Authority (TVA) Docket Nos. 50-327 and 50-328, Sequoyah Nuclear Plant (SQN), Units 1 and 2, Hamilton County, Tennessee
TVA Docket Nos. 50-390 and 50-391, Watts Bar Nuclear Plant (WBN), Units 1 and 2, Rhea County, Tennessee

Date of amendment request: August 7, 2017.

Brief description of amendments: The amendments revised technical specifications (TSs) limiting conditions for operation and surveillance requirements related to the reactor trip system instrumentation for all four units.

Date of issuance: August 30, 2018.

Effective date: As of the date of issuance and shall be implemented in 30 days.

Amendment Nos.: SQN, 343 (Unit 1) and 336 (Unit 2); and WBN, 122 (Unit 1) and 21 (Unit 2). A publicly-available version is in ADAMS under Accession No. ML18197A307; documents related to these amendments are listed in the Safety Evaluation (SE) enclosed with the amendments.

Renewed Facility Operating License Nos. DPR-77, DPR-79 and Facility Operating License Nos. NPF-90 and NPF-96: Amendments revised the Facility Operating Licenses and TSs.

Date of initial notice in Federal Register: November 21, 2017 (82 FR 55416).

The Commission's related evaluation of the amendments is contained in SE dated August 30, 2018.

No significant hazards consideration comments received: No.

Dated at Rockville, Maryland, on September 14, 2018.

For the Nuclear Regulatory Commission.

Kathryn M. Brock,
Acting Director, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.

[FR Doc. 2018-20333 Filed 9-24-18; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[NRC-2018-0001]

Sunshine Act Meetings

TIME AND DATE: Weeks of September 24, October 1, 8, 15, 22, 29, 2018.

PLACE: Commissioners' Conference Room, 11555 Rockville Pike, Rockville, Maryland.

STATUS: Public and closed.

MATTERS TO BE CONSIDERED:

Week of September 24, 2018

Thursday, September 27, 2018

10:00 a.m. Strategic Programmatic Overview of the Operating Reactors Business Line (Public) (Contact: Trent Wertz: 301-415-1568).

This meeting will be webcast live at the Web address—<http://www.nrc.gov/>.

Week of October 1, 2018—Tentative

There are no meetings scheduled for the week of October 1, 2018.

Week of October 8, 2018—Tentative

Thursday, October 11, 2018

9:00 a.m. Strategic Programmatic Overview of the Decommissioning and Low-Level Waste and Spent Fuel Storage and Transportation Business Lines (Public) (Contact: Matthew Meyer: 301-415-6198).

This meeting will be webcast live at the Web address—<http://www.nrc.gov/>.

Week of October 15, 2018—Tentative

There are no meetings scheduled for the week of October 15, 2018.

Week of October 22, 2018—Tentative

Thursday, October 25, 2018

9:00 a.m. Briefing on Digital Instrumentation and Control (Public) (Contact: Jason Paige: 301-415-1474).

This meeting will be webcast live at the Web address—<http://www.nrc.gov/>.

Week of October 29, 2018—Tentative

There are no meetings scheduled for the week of October 29, 2018.

CONTACT PERSON FOR MORE INFORMATION:

For more information or to verify the status of meetings, contact Denise McGovern at 301-415-0681 or via email at Denise.McGovern@nrc.gov. The schedule for Commission meetings is subject to change on short notice.

The NRC Commission Meeting Schedule can be found on the internet at: <http://www.nrc.gov/public-involve/public-meetings/schedule.html>.

The NRC provides reasonable accommodation to individuals with disabilities where appropriate. If you need a reasonable accommodation to participate in these public meetings, or need this meeting notice or the transcript or other information from the public meetings in another format (e.g., braille, large print), please notify Kimberly Meyer-Chambers, NRC Disability Program Manager, at 301-287-0739, by videophone at 240-428-3217, or by email at Kimberly.Meyer-Chambers@nrc.gov. Determinations on requests for reasonable accommodation will be made on a case-by-case basis.

Members of the public may request to receive this information electronically. If you would like to be added to the distribution, please contact the Nuclear Regulatory Commission, Office of the Secretary, Washington, DC 20555 (301–415–1969), or you may email Patricia.Jimenez@nrc.gov or Wendy.Moore@nrc.gov.

Dated at Rockville, Maryland, this 20th day of September, 2018.

For the Nuclear Regulatory Commission.

Denise L. McGovern,
Policy Coordinator, Office of the Secretary.
[FR Doc. 2018–20888 Filed 9–21–18; 11:15 am]
BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

RIN 3150–AI01

[NRC–2014–0137]

Regulatory Guidance and Technical Basis on the Alternate Pressurized Thermal Shock Rule

AGENCY: Nuclear Regulatory Commission.

ACTION: Regulatory guide; issuance.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) is issuing Revision 0 to Regulatory Guide (RG) 1.230, “Regulatory Guidance on the Alternate Pressurized Thermal Shock Rule” and the associated technical basis, NUREG–2163, “Technical Basis for Regulatory Guidance on the Alternate Pressurized Thermal Shock Rule.” This guidance describes a method that the NRC staff considers acceptable to permit use of the alternate fracture toughness requirements for protection against pressurized thermal shock events for pressurized water reactor pressure vessels.

DATES: Revision 0 to RG 1.230 is available on September 25, 2018.

ADDRESSES: Please refer to Docket ID NRC–2014–0137 when contacting the NRC about the availability of information regarding this document. You may obtain publically-available information related to this document, using the following methods:

- *Federal Rulemaking Website:* Go to <http://www.regulations.gov> and search for Docket ID NRC–2014–0137. Address questions about docket IDs in *Regulations.gov* to Jennifer Borges; telephone: 301–287–9127; email: Jennifer.Borges@nrc.gov. For technical questions, contact the individuals listed in the **FOR FURTHER INFORMATION CONTACT** section of this document.

- *NRC’s Agencywide Documents Access and Management System*

(ADAMS): You may obtain publicly-available documents online in the ADAMS Public Document collection at <http://www.nrc.gov/reading-rm/adams.html>. To begin the search, select “Begin Web-based ADAMS Search.” For problems with ADAMS, please contact the NRC’s Public Document Room (PDR) reference staff at 1–800–397–4209, 301–415–4737, or by email to pdr.resource@nrc.gov. The ADAMS accession number for each document referenced in this notice (if that document is available in ADAMS) is provided the first time that a document is referenced. Revision 0 to Regulatory Guide 1.230, and the regulatory analysis may be found in ADAMS under Accession No. ML15344A402 and ML14056A013 respectively. Revision 0 to NUREG–2163 may be found in ADAMS under Accession No. ML18255A118.

- *NRC’s PDR:* You may examine and purchase copies of public documents at the NRC’s PDR, Room O1–F21, One White Flint North, 11555 Rockville Pike, Rockville, Maryland 20852.

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FOR FURTHER INFORMATION CONTACT:

Mark Kirk, Office of Nuclear Regulatory Research, telephone: 301–415–2456, email: Mark.Kirk@nrc.gov, and Stephen Burton, Office of Nuclear Regulatory Research, telephone: 301–415–7000, email: Stephen.Burton@nrc.gov. Both are staff of the U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001.

SUPPLEMENTARY INFORMATION:

I. Introduction

The NRC is issuing a new guide in the NRC’s “Regulatory Guide” series. This series was developed to describe and make available to the public information regarding methods that are acceptable to the NRC staff for implementing specific parts of the agency’s regulations, techniques that the NRC staff uses in evaluating specific issues or postulated events, and data that the NRC staff needs in its review of applications for permits and licenses.

Revision 0 of RG 1.230 was issued with a temporary identification of Draft Regulatory Guide, DG–1299. This RG is being issued to describe a method that the staff of the NRC considers acceptable to meet the alternate fracture toughness requirements for protection against pressurized thermal shock (PTS) events for pressurized water reactor (PWR) reactor pressure vessels (RPVs) in section 50.61a of title 10 of the *Code of Federal Regulations* (10 CFR), “Alternate Fracture Toughness

Requirements for Protection against Pressurized Thermal Shock Events.” The alternate PTS requirements are based on updated analysis methods, whereas the requirements in 10 CFR 50.61, “Fracture Toughness Requirements for Protection against Pressurized Thermal Shock Events,” are based on overly conservative probabilistic fracture mechanics analyses.

NUREG–2163 explains the basis for the requirements that establish the threshold conditions to permit use of 10 CFR 50.61a in lieu of 10 CFR 50.61 and describes methods by which the following four requirements can be met: (1) Criteria relating to the date of construction and design requirements, (2) criteria relating to evaluation of plant specific surveillance data, (3) criteria relating to inservice inspection data and non-destructive examination requirements, and (4) criteria relating to alternate limits on embrittlement. Revision 0 of RG 1.230 also describes these methods in a manner consistent with NUREG–2163.

II. Additional Information

The DG–1299 and draft NUREG–2163 were published in the **Federal Register** on March 13, 2015 (80 FR 13449) for a 60-day public comment period. The public comment period closed on May 12, 2015. Public comments on DG–1299 and draft NUREG–2163 and the NRC staff responses to the public comments are available in ADAMS under Accession No. ML15344A398.

III. Congressional Review Act

RG 1.230 is a rule as defined in the Congressional Review Act (5 U.S.C. 801–808). However, the Office of Management and Budget has not found it to be a major rule as defined in the Congressional Review Act.

IV. Backfitting and Issue Finality

The RG 1.230 provides guidance on the methods acceptable to the NRC for complying with the NRC’s regulations associated with alternate fracture toughness requirements for protection against PTS events for PWR RPVs. NUREG–2163 provides technical bases that support the RG. The RG, like the alternate PTS requirements of 10 CFR 50.61a, applies to holders of operating licenses for a PWR whose construction permit was issued before February 3, 2010.

Issuance of this RG and NUREG does not constitute backfitting under 10 CFR part 50. As discussed in the “Implementation” section of RG 1.230, the NRC has no current intention to impose the RG on current holders of 10