

containment pressure and assuring that leakage through the primary containment does not exceed allowable leakage rate values.

The proposed exemption would allow RBS to continue to use an alternate definition of P_a of 7.6 psig. This use of the alternate definition for P_a meets the intent of 10 CFR part 50, appendix J because it provides testing of the primary containment parameters at a bounding pressure that is calculated to be possible throughout containment over a sustained period following a design basis accident.

The Need for the Proposed Action

The proposed exemption is needed to allow RBS to continue to use an alternate definition for P_a which results in the use of a lower pressure for appendix J containment testing. Use of the lower pressure reduces the burden of modifying the test procedures, seeking NRC authorization to change the current TS value, and conducting the testing at the higher pressure. In addition, applying the literal definition for P_a would not serve the underlying purpose of the rule which is to test the primary containment parameters at a peak pressure calculated to exist over the long term following a design basis accident.

Environmental Impacts of the Proposed Action

The NRC has completed its evaluation of the proposed action and concludes that the exemption described above meets the intended purpose of the requirements in 10 CFR part 50, appendix J. The lower calculated P_a value provides a representative bounding pressure for evaluating the leak-tight integrity of the primary reactor containment and related penetrations.

The proposed action will not significantly increase the probability or consequences of accidents. No changes are being made in the types of effluents that may be released offsite. There is no significant increase in the amount of any effluent released offsite. There is no significant increase in occupational or public radiation exposure. Therefore, there are no significant radiological environmental impacts associated with the proposed action.

With regard to potential non-radiological impacts, the proposed action does not have any foreseeable impacts to land, air, or water resources, including impacts to biota. In addition, there are also no known socioeconomic or environmental justice impacts associated with such proposed action. Therefore, there are no significant non-

radiological environmental impacts associated with the proposed action.

Accordingly, the NRC concludes that there are no significant environmental impacts associated with the proposed action.

Environmental Impacts of the Alternatives to the Proposed Action

As an alternative to the proposed action, the staff considered denial of the proposed action (i.e., the “no-action” alternative). Denial of the application would result in no change in current environmental impacts. The environmental impacts of the proposed action and the alternative action are similar.

Alternative Use of Resources

The action does not involve the use of any different resources than those previously considered in the “Final Environmental Statement,” NUREG-1073, January 1985, for the RBS.

Agencies and Persons Notified

In accordance with its stated policy, on August 6, 2013, the staff notified the Louisiana State official, Ji Wiley, of the Louisiana Department of Environmental Quality, Radiation Protection Division, regarding the environmental impact of the proposed action. The State official had no comments.

III. Finding of No Significant Impact

On the basis of the environmental assessment, the NRC concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the NRC has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee's letter dated August 23, 2012.

Dated at Rockville, Maryland, this 9th day of August 2013. For the Nuclear Regulatory Commission.

Michael T. Markley,

Chief, Plant Licensing Branch IV, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.

[FR Doc. 2013-20146 Filed 8-16-13; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 52-025, and 52-026; NRC-2008-0252]

Vogtle Electric Generating Station, Units 3 and 4; Southern Nuclear Operating Company; Changes to the Chemical Volume Control System

AGENCY: Nuclear Regulatory Commission.

ACTION: Exemption and combined license amendment; issuance.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) is both granting an exemption to allow a departure from the certification information of Tier 1 of the generic design control document (DCD) and issuing License Amendment No. 12 to Combined Licenses (COL), NPF-91 and NPF-92. The COLs were issued to Southern Nuclear Operating Company, Inc., and Georgia Power Company, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and the City of Dalton, Georgia (the licensee), for construction and operation of the Vogtle Electric Generating Plant (VEGP), Units 3 and 4, located in Burke County, Georgia. The amendment requests changes that modify the Chemical and Volume Control System (CVS), including changes to information located in Tier 1 Tables 2.3.2-1 and 2.3.2-2, and Tier 1 Figures 2.2.1-1 and 2.3.2-1. The granting of the exemption allows the changes to Tier 1 information as specified in the license amendment request. Because the acceptability of the exemption was determined in part by the acceptability of the amendment, the exemption and amendment are being issued concurrently.

ADDRESSES: Please refer to Docket ID NRC-2008-0252 when contacting the NRC about the availability of information regarding this document. You may access information related to this document, which the NRC possesses and is publicly available, using any of the following methods:

- Federal Rulemaking Web site: Go to <http://www.regulations.gov> and search for Docket ID NRC-2008-0252. Address questions about NRC dockets to Carol Gallagher; telephone: 301-287-3422; email: Carol.Gallagher@nrc.gov. For technical questions, contact the individual listed in the **FOR FURTHER INFORMATION CONTACT** section of this document.

- NRC's Agencywide Documents Access and Management System (ADAMS): You may access publicly available documents online in the NRC Library at <http://www.nrc.gov/reading-rm/adams.html>. To begin the search,

select “ADAMS Public Documents” and then select “Begin Web-based ADAMS Search.” For problems with ADAMS, please contact the NRC’s Public Document Room (PDR) reference staff at 1–800–397–4209, 301–415–4737, or by email to pdr.resource@nrc.gov. The ADAMS accession number for each document referenced in this document (if that document is available in ADAMS) is provided the first time that a document is referenced. The request for the amendment and exemption were submitted by letter dated January 11, 2013 (ADAMS Accession No. ML13016A091). The licensee supplemented this request on February 27, 2013 (ADAMS Accession No. ML13059A499) and June 11, 2013 (ADAMS Accession No. ML13163A356).

- NRC’s PDR: You may examine and purchase copies of public documents at the NRC’s PDR, Room O1–F21, One White Flint North, 11555 Rockville Pike, Rockville, Maryland 20852.

FOR FURTHER INFORMATION CONTACT:

Anthony Minarik, Office of New Reactors, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001; telephone: 301–415–6185; email: Anthony.Minarik@nrc.gov.

SUPPLEMENTARY INFORMATION:

I. Introduction

The NRC is issuing an exemption from Paragraph B of Section III, “Scope and Contents,” of Appendix D, “Design Certification Rule for the AP1000,” to part 52 of Title 10 of the *Code of Federal Regulations* (10 CFR), and License Amendment No. 12 to COLs, NPF–91 and NPF–92, issued to the licensee. The exemption is required by Paragraph A.4 of Section VIII, “Processes for Changes and Departures,” Appendix D to 10 CFR Part 52 to allow the licensee to depart from Tier 1 information. With the requested amendment, the licensee sought to modify the design of the CVS. As part of this request, the licensee needed to change information located in Tier 1 Tables 2.3.2–1 and 2.3.2–2, and Tier 1 Figures 2.2.1–1 and 2.3.2–1. These changes were necessary as part of a design modification which provides a spring-assisted check valve to the Reactor Coolant System Purification Return Line in order to maintain overpressure protection, replaces an isolation check valve in the CVS with an air operated globe valve, and separates the zinc and hydrogen injection lines.

Part of the justification for granting the exemption was provided by the review of the amendment. Because the exemption is necessary in order to issue the requested license amendment, the NRC granted the exemption and issued

the amendment concurrently, rather than in sequence. This included issuing a combined safety evaluation containing the NRC staff’s review of both the exemption request and the license amendment. The exemption met all applicable regulatory criteria set forth in 10 CFR 50.12, 10 CFR 52.7, and Section VIII.A.4. of Appendix D to 10 CFR Part 52. The license amendment was found to be acceptable as well. The combined safety evaluation is available in ADAMS under Accession No. ML13172A326.

Identical exemption documents (except for referenced unit numbers and license numbers) were issued to the licensee for Vogtle Units 3 and 4 (COLs NPF–91 and NPF–92). These documents can be found in ADAMS under Accession Nos. ML13172A218 and ML13172A223. The exemption is reproduced (with the exception of abbreviated titles and additional citations) in Section II of this document. The amendment documents for COLs NPF–91 and NPF–92 are available in ADAMS under Accession Nos. ML13172A242 and ML13172A299. A summary of the amendment documents is provided in Section III of this document.

II. Exemption

Reproduced below is the exemption document issued to Vogtle Unit 3 and Unit 4. It makes reference to the combined safety evaluation that provides the reasoning for the findings made by the NRC (and listed under Item 1) in order to grant the exemption:

1. In a letter dated January 11, 2013, and as supplemented by letters dated February 27, 2013 and June 11, 2013, the licensee requested from the Commission an exemption from the provisions of 10 CFR part 52, appendix D, Section III.B, as part of license amendment request 13–002, “Changes to the Chemical and Volume Control System (CVS) (LAR–13–002).”

For the reasons set forth in Section 3.1, “Evaluation of Exemption,” of the NRC staff’s Safety Evaluation, which can be found in ADAMS under Accession No. ML13172A326, the Commission finds that:

A. The exemption is authorized by law;

B. the exemption presents no undue risk to public health and safety;

C. the exemption is consistent with the common defense and security;

D. special circumstances are present in that the application of the rule in this circumstance is not necessary to serve the underlying purpose of the rule;

E. the special circumstances outweigh any decrease in safety that may result

from the reduction in standardization caused by the exemption; and

F. the exemption will not result in a significant decrease in the level of safety otherwise provided by the design.

2. Accordingly, the licensee is granted an exemption to the provisions of 10 CFR part 52, appendix D, Section III.B, to allow deviations from the certified DCD Tier 1 Tables 2.3.2–1 and 2.3.2–2, and Figures 2.2.1–1 and 2.3.2–1, as described in the licensee’s request dated January 11, 2013, and as supplemented on February 27, 2013 and June 11, 2013. This exemption is related to, and necessary for the granting of License Amendment No. 12, which is being issued concurrently with this exemption.

3. As explained in Section 5.0, “Environmental Consideration,” of the NRC staff’s Safety Evaluation (ADAMS Accession No. ML13172A326), this exemption meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Therefore, pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment needs to be prepared in connection with the issuance of the exemption.

4. This exemption is effective as of July 25, 2013.

III. License Amendment Request

By letter dated January 11, 2013, the licensee requested that the NRC amend the COLs for VEGP, Units 3 and 4, COLs NPF–91 and NPF–92. The licensee supplemented this application on February 27, 2013, and June 11, 2013. The proposed amendment would depart from the UFSAR Tier 1 material, and would revise the associated material that has been included in Appendix C of each of the VEGP, Units 3 and 4, COLs. The requested amendment will revise the Tier 2 UFSAR information pertaining to the CVS, found throughout the UFSAR. These Tier 2 changes require modifications to particular information located in Tier 1 Tables 2.3.2–1 and 2.3.2–2, and Tier 1 Figures 2.2.1–1 and 2.3.2–1. These changes were necessary as part of a design modification which provides a spring-assisted check valve to the Reactor Coolant System Purification Return Line in order to maintain overpressure protection, replaces an isolation check valve in the CVS with an air operated globe valve, and separates the zinc and hydrogen injection lines.

The Commission has determined for these amendments that the application complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission’s rules and regulations.

The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

A notice of consideration of issuance of amendment to facility operating license or combined license, as applicable, proposed no significant hazards consideration determination, and opportunity for a hearing in connection with these actions, was published in the **Federal Register** on March 4, 2013 (78 FR 14137). The supplements had no effect on the no significant hazards consideration determination and no comments were received during the 60-day comment period.

The Commission has determined that these amendments satisfy the criteria for categorical exclusion in accordance with 10 CFR 51.22. Therefore, pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared for these amendments.

IV. Conclusion

Using the reasons set forth in the combined safety evaluation, the staff granted the exemption and issued the amendment that the licensee requested on January 11, 2013, and supplemented by letters February 27, 2013 and June 11, 2013. The exemption and amendment were issued on July 25, 2013 as part of a combined package to the licensee. (ADAMS Accession No. ML13172A181).

Dated at Rockville, Maryland, this 12th day of August, 2013.

For the Nuclear Regulatory Commission,
Lawrence Burkhardt,

Chief, Licensing Branch 4, Division of New Reactor Licensing, Office of New Reactors.

[FR Doc. 2013-20144 Filed 8-16-13; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards (ACRS) Meeting of the ACRS Subcommittee on Digital I&C; Notice of Meeting

The ACRS Subcommittee on Digital I&C will hold a briefing on September 19, 2013, Room T-2B3, 11545 Rockville Pike, Rockville, Maryland.

The entire meeting will be open to public attendance.

The agenda for the subject meeting shall be as follows:

Thursday, September 19, 2013-8:30 a.m. until 5:00 p.m.

The Subcommittee will review the results and status of NRC research activities involving identification of digital system failure modes and use of hazard analysis methods for digital safety systems. The Subcommittee will hear presentations by and hold discussions with the Electric Power Research Institute (EPRI), the NRC staff, and other interested persons regarding this matter. The Subcommittee will gather information, analyze relevant issues and facts, and formulate proposed positions and actions, as appropriate, for deliberation by the Full Committee.

Members of the public desiring to provide oral statements and/or written comments should notify the Designated Federal Official (DFO), Christina Antonescu (Telephone 301-415-6792 or Email: Christina.Antonescu@nrc.gov) five days prior to the meeting, if possible, so that appropriate arrangements can be made. Thirty-five hard copies of each presentation or handout should be provided to the DFO thirty minutes before the meeting. In addition, one electronic copy of each presentation should be emailed to the DFO one day before the meeting. If an electronic copy cannot be provided within this timeframe, presenters should provide the DFO with a CD containing each presentation at least thirty minutes before the meeting. Electronic recordings will be permitted only during those portions of the meeting that are open to the public. Detailed procedures for the conduct of and participation in ACRS meetings were published in the **Federal Register** on October 18, 2012, (77 FR 64146-64147).

Detailed meeting agendas and meeting transcripts are available on the NRC Web site at <http://www.nrc.gov/reading-rm/doc-collections/acrs>. Information regarding topics to be discussed, changes to the agenda, whether the meeting has been canceled or rescheduled, and the time allotted to present oral statements can be obtained from the Web site cited above or by contacting the identified DFO. Moreover, in view of the possibility that the schedule for ACRS meetings may be adjusted by the Chairman as necessary to facilitate the conduct of the meeting, persons planning to attend should check with these references if such rescheduling would result in a major inconvenience.

If attending this meeting, please enter through the One White Flint North building, 11555 Rockville Pike,

Rockville, MD. After registering with security, please contact Mr. Theron Brown (Telephone 240-888-9835) to be escorted to the meeting room.

Dated: August 1, 2013.

Cayetano Santos,

Chief, Technical Support Branch, Advisory Committee on Reactor Safeguards.

[FR Doc. 2013-20142 Filed 8-16-13; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards (ACRS); Meeting of the ACRS Subcommittee on Fukushima; Notice of Meeting

The ACRS Subcommittee on Fukushima will hold a meeting on September 4, 2013, Room T-2B1, 11545 Rockville Pike, Rockville, Maryland.

The entire meeting will be open to public attendance.

The agenda for the subject meeting shall be as follows:

Wednesday, September 4, 2013-8:30 a.m. until 12:00 p.m.

The Subcommittee will review and discuss options for addressing the Near Term Task Force (NTTF) Recommendation 1: Enhanced Regulatory Framework. The Subcommittee will hear presentations by and hold discussions with the NRC staff, the Nuclear Energy Institute, and other interested persons regarding this matter. The Subcommittee will gather information, analyze relevant issues and facts, and formulate proposed positions and actions, as appropriate, for deliberation by the Full Committee.

Members of the public desiring to provide oral statements and/or written comments should notify the Designated Federal Official (DFO), Hossein Nourbakhsh (Telephone 301-415-5622 or Email: Hossein.Nourbakhsh@nrc.gov) five days prior to the meeting, if possible, so that appropriate arrangements can be made. Thirty-five hard copies of each presentation or handout should be provided to the DFO thirty minutes before the meeting. In addition, one electronic copy of each presentation should be emailed to the DFO one day before the meeting. If an electronic copy cannot be provided within this timeframe, presenters should provide the DFO with a CD containing each presentation at least thirty minutes before the meeting. Electronic recordings will be permitted only during those portions of the meeting that are open to the public. Detailed procedures for the conduct of