Dated: August 15, 2000.

Matthew M. Crouch,

Advisory Committee Management Officer, National Aeronautics and Space Administration.

[FR Doc. 00-21170 Filed 8-18-00; 8:45 am]

BILLING CODE 7510-01-P

## NATIONAL AERONAUTICS AND SPACE ADMINISTRATION

[Notice 00-094]

### Centennial of Flight Commission

**AGENCY:** National Aeronautics and Space Administration.

**ACTION:** Notice of meeting.

SUMMARY: In accordance with the Federal Advisory Committee Act, Pub. L. 92-463, as amended, the National Aeronautics and Space Administration announces a meeting of the Centennial of Flight Commission.

DATES: Tuesday, September 12, 2000, 1 p.m. to 3:30 p.m.

ADDRESSES: Smithsonian National Air and Space Museum, 7th and Independence Avenue, SW, Director's Conference Room, 3rd Floor, Washington, DC 20560. Attendees must check in at the Information Desk to be cleared to the 3rd floor.

FOR FURTHER INFORMATION CONTACT: Ms. Beverly Farmarco, Code ZC, National Aeronautics and Space Administration, Washington, DC 20546, 202/358-1903.

SUPPLEMENTARY INFORMATION: The meeting will be open to the public up to the seating capacity of the room. The agenda for the meeting is as follows:

- -Experimental Aircraft Association Plans for the Wright Centennial
- -Discussion of Chair's Draft "White Paper on a Strategy Posture for the US Centennial of Flight Commission"
- -Discussion of FY 2000 Report to Congress
- —Plans for Next Meeting

It is imperative that the meeting be held on this date to accommodate the scheduling priorities of the key participants. Visitors will be requested to sign a visitor's register.

Dated: August 15, 2000.

#### Matthew M. Crouch

Advisory Committee Management Officer, National Aeronautics and Space Administration.

[FR Doc. 00-21169 Filed 8-18-00; 8:45 am] BILLING CODE 7510-01-P

### **NUCLEAR REGULATORY** COMMISSION

[Docket No. 50-412]

Pennsylvania Power Company, Ohio Edison Company, The Cleveland **Electric Illuminating Company, The** Toledo Edison Company, Firstenergy **Nuclear Operating Company; Beaver** Valley Power Station, Unit No. 2; **Environmental Assessment and Finding of No Significant Impact** 

The Nuclear Regulatory Commission (NRC) is considering issuance of an exemption from the requirements of title 10 of the Code of Federal Regulations (10 CFR) § 50.60(a), and 10 CFR part 50, Appendix G, for Facility Operating License No. NPF-73, issued to FirstEnergy Nuclear Operating Company (the licensee), for operation of the Beaver Valley Power Station, Unit No. 2 (BVPS-2), located in Beaver County, Pennsylvania.

#### **Environmental Assessment**

Identification of the Proposed Action

Appendix G to 10 CFR part 50, requires that pressure/temperature (P/T) limits be established for reactor pressure vessels during normal operating and hydrostatic or leak rate testing conditions. Specifically, this regulation states, "The appropriate requirements on both the pressure-temperature limits and the minimum permissible temperature must be met for all conditions." Additionally, it specifies that the requirements for these limits are contained in the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (Code), Section XI, Appendix G.

To address provisions of an amendment to the Technical Specification P/T limits, the licensee requested in its submittal dated June 17, 1999, that the NRC staff exempt BVPS-2 from the requirements of 10 CFR part 50, § 50.60(a), and 10 CFR part 50, Appendix G, to allow application of ASME Code Case N-640 in establishing the reactor vessel pressure limits at low temperatures.

Code Case N-640 permits the use of an alternate reference fracture toughness (K<sub>IC</sub> fracture toughness curve instead of the  $K_{Ia}$  fracture toughness curve) for reactor vessel materials in determining the P/T limits. Since the  $K_{\rm IC}$  fracture toughness curve shown in ASME, Section XI, Appendix A, Figure A-2200–1 (the K<sub>IC</sub> fracture toughness curve), provides greater allowable fracture toughness than the corresponding K<sub>Ia</sub> fracture toughness curve of ASME, Section XI, Appendix

G, Figure G-2210-1 (the  $K_{Ia}$  fracture toughness curve), using Code Case N-640 for establishing the P/T limits would be less conservative than the methodology currently endorsed by 10 CFR part 50, Appendix G. Therefore, an exemption is required in order to apply the Code Case. It should be noted that, although Code Case N-640 was incorporated into the ASME Code recently, an exemption is still required because the proposed P/T limits (excluding Code Case N-640) are based on the 1989 edition of the ASME Code.

The proposed action is in accordance with the licensee's application for exemption dated June 17, 1999.

The Need for the Proposed Action

ASME Code Case N-640 is needed to revise the method used to determine the reactor coolant system (RCS) P/T limits.

The purpose of 10 CFR part 50, § 50.60(a), and 10 CFR part 50, Appendix G, is to protect the integrity of the reactor coolant pressure boundary in nuclear power plants. This is accomplished through these regulations that, in part, specify fracture toughness requirements for ferritic materials of the reactor coolant pressure boundary. Pursuant to 10 CFR part 50, Appendix G, it is required that P/T limits for the RCS be at least as conservative as those obtained by applying the methodology of the ASME Code, Section XI, Appendix G.

Current overpressure protection system (OPPS) setpoints produce operational constraints by limiting the P/T range available to the operator to heat up or cool down the plant. The operating window through which the operator heats up and cools down the RCS becomes more restrictive with continued reactor vessel service. Reducing this operating window could potentially have an adverse safety impact by increasing the possibility of inadvertent OPPS actuation due to pressure surges associated with normal plant evolutions such as reactor coolant pump start and swapping operating charging pumps with the RCS in a water-solid condition. The impact on the P/T limits and OPPS setpoints has been evaluated for an increased service period to 15 effective full power years based on ASME Code, Section XI, Appendix G, requirements. The results indicate that OPPS would significantly restrict the ability to perform plant heatup and cooldown, create an unnecessary burden to plant operations, and challenge control of plant evolutions required with OPPS enabled. Continued operation of BVPS-2 with P/ T curves developed to satisfy ASME Code, Section XI, Appendix G,

requirements without the relief provided by ASME Code Case N-640 would unnecessarily restrict the P/T operating window, especially at low

temperature conditions.

Application of ASME Code Case N-640 will provide results which are sufficiently conservative to ensure the integrity of the reactor coolant pressure boundary while providing P/T curves which are not overly restrictive. Implementation of the proposed P/T curves, as allowed by ASME Code Case N-640, does not significantly reduce the margin of safety.

In the associated exemption, the NRC staff has determined that, pursuant to 10 CFR 50.12(a)(2)(ii), the underlying purpose of the regulation will continue to be served by the implementation of

ASME Code Case N-640.

Environmental Impacts of the Proposed Action

The NRC has completed its evaluation of the proposed action and concludes that the proposed action provides adequate margin of safety against brittle failure of the reactor coolant pressure

The proposed action will not significantly increase the probability or consequences of accidents, no changes are being made in the types of any effluents that may be released off site, and there is no significant increase in occupational or public radiation exposure. Therefore, there are no significant radiological environmental impacts associated with the proposed action.

With regard to potential nonradiological impacts, the proposed action does not involve any historic sites. It does not affect nonradiological plant effluents and has no other environmental impact. Therefore, there are no significant nonradiological environmental impacts associated with the proposed action.

Accordingly, the NRC concludes that there are no significant environmental impacts associated with the proposed action.

Alternatives to the Proposed Action

As an alternative to the proposed action, the staff considered denial of the proposed action (i.e., the "no-action" alternative). Denial of the application would result in no change in current environmental impacts. The environmental impacts of the proposed action and the alternative action are similar.

Alternative Use of Resources

This action does not involve the use of any resources not previously

considered in the Final Environmental Statement for BVPS-2.

Agencies and Persons Consulted

In accordance with its stated policy, on July 10, 2000, the staff consulted with the Pennsylvania State official, Mr. L. Ryan of the Pennsylvania Department of Environmental Protection Bureau, Division of Nuclear Safety, regarding the environmental impact of the proposed action. The State official had no comments.

## **Finding of No Significant Impact**

On the basis of the environmental assessment, the NRC concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the NRC has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee's letter dated June 17, 1999, which is available for public inspection at the Commission's Public Document Room, The Gelman Building, 2120 L Street, NW., Washington, DC. Publicly available records will be accessible electronically from the ADAMS Public Library component on the NRC Web site, http://www.nrc.gov (the Electronic Reading Room).

Dated at Rockville, Maryland, this 16th day of August, 2000.

For the Nuclear Regulatory Commission. Daniel S. Collins,

Project Manager, Section 1, Project Directorate I, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.

[FR Doc. 00-21230 Filed 8-18-00; 8:45 am] BILLING CODE 7590-01-P

## OFFICE OF PERSONNEL **MANAGEMENT**

# **Federal Prevailing Rate Advisory Committee; Open Committee Meetings**

According to the provisions of section 10 of the Federal Advisory Committee Act (Pub. L. 92–463), notice is hereby given that meetings of the Federal Prevailing Rate Advisory Committee will be held on-

Thursday, September 7, 2000 Thursday, September 21, 2000 Thursday, September 28, 2000

The meeting will start at 10 a.m. and will be held in Room 5A06A, Office of Personnel Management Building, 1900 E Street, NW., Washington, DC.

The Federal Prevailing Rate Advisory Committee is composed of a Chair, five

representatives from labor unions holding exclusive bargaining rights for Federal blue-collar employees, and five representatives from Federal agencies. Entitlement to membership on the Committee is provided for in 5 U.S.C. 5347.

The Committee's primary responsibility is to review the Prevailing Rate System and other matters pertinent to establishing prevailing rates under subchapter IV, chapter 53, 5 U.S.C., as amended, and from time to time advise the Office of Personnel Management.

This scheduled meeting will start in open session with both labor and management representatives attending. During the meeting either the labor members or the management members may caucus separately with the Chair to devise strategy and formulate positions. Premature disclosure of the matters discussed in these caucuses would unacceptably impair the ability of the Committee to reach a consensus on the matters being considered and would disrupt substantially the disposition of its business. Therefore, these caucuses will be closed to the public because of a determination made by the Director of the Office of Personnel Management under the provisions of section 10(d) of the Federal Advisory Committee Act (Pub. L. 92-463) and 5 U.S.C. 552b(c)(9)(B). These caucuses may, depending on the issues involved, constitute a substantial portion of a meeting.

Annually, the Chair compiles a report of pay issues discussed and concluded recommendations. These reports are available to the public, upon written request to the Committee's Secretary.

The public is invited to submit material in writing to the Chair on Federal Wage System pay matters felt to be deserving of the Committee's attention. Additional information on this meeting may be obtained by contacting the Committee's Secretary, Office of Personnel Management, Federal Prevailing Rate Advisory Committee, Room 5538, 1900 E Street, NW., Washington, DC 20415, (202) 606-1500.

Dated: August 10, 2000.

## John F. Levden,

Chairman, Federal Prevailing Rate Advisory Committee.

[FR Doc. 00-21176 Filed 8-18-00; 8:45 am] BILLING CODE 6325-01-P

## **POSTAL SERVICE**

## Plan for Secure Postage Meter Technology

**AGENCY:** Postal Service.