

worker to sit down or be provided with a stretcher and pulled across the undercast to either the inby or outby ends. The petitioner asserts that the proposed alternative method will provide at least the same measure of protection as the mandatory standard.

22. Alex Energy, Inc.

[Docket No. M-1999-105-C]

Alex Energy, Inc., P.O. Box 857, Summersville, West Virginia 26651 has filed a petition to modify the application of 30 CFR 75.1700 (oil and gas wells) to its Jerry Fork Eagle Mine (I.D. No. 46-08787) located in Nicholas County, West Virginia. The petitioner proposes to plug and mine through oil and gas wells and to notify the District Manager or designee prior to mining within 300 feet of the well. The petitioner asserts that the proposed alternative method will provide at least the same measure of protection as the mandatory standard.

Request for Comments

Persons interested in these petitions are encouraged to submit comments via e-mail to "comments@msha.gov," or on a computer disk along with an original hard copy to the Office of Standards, Regulations, and Variances, Mine Safety and Health Administration, 4015 Wilson Boulevard, Room 627, Arlington, Virginia 22203. All comments must be postmarked or received in that office on or before November 26, 1999. Copies of these petitions are available for inspection at that address.

Dated: October 18, 1999.

Carol J. Jones,

Acting Director, Office of Standards, Regulations, and Variances.

[FR Doc. 99-27855 Filed 10-25-99; 8:45 am]

BILLING CODE 4510-43-P

NATIONAL TRANSPORTATION SAFETY BOARD

Sunshine Act Meeting; Agenda

TIME AND DATE: 9:30 a.m., Tuesday, November 2, 1999.

PLACE: NTSB Board Room, 5th Floor, 490 L'Enfant Plaza, S.W., Washington, D.C. 20594.

STATUS: Open to the Public.

MATTERS TO BE CONSIDERED:

6952A Aviation Accident Report: Controlled flight into Terrain, Korean Air Flight 801, Boeing 747-300, HL7468, Nimitz Hill, Guam, on August 6, 1997

NEWS MEDIA CONTACT: Telephone: (202) 314-6100.

Individuals requesting specific accommodation should contact Mrs. Barbara Bush at (202) 314-6220 by Friday, October 29, 1999.

FOR MORE INFORMATION CONTACT: Rhonda Underwood (202) 314-6065.

Dated: October 22, 1999.

Rhonda Underwood,

Federal Register Liaison Officer.

[FR Doc. 99-28097 Filed 10-22-99; 3:51 pm]

BILLING CODE 7533-01-M

NUCLEAR REGULATORY COMMISSION

Agency Information Collection Activities: Submission for OMB Review; Comment Request

AGENCY: U.S. Nuclear Regulatory Commission (NRC).

ACTION: Notice of the OMB review of information collection and solicitation of public comment.

SUMMARY: The NRC has recently submitted to OMB for review the following proposal for the collection of information under the provisions of the Paperwork Reduction Act of 1995 (44 U.S.C. Chapter 35). The NRC hereby informs potential respondents that an agency may not conduct or sponsor, and that a person is not required to respond to, a collection of information unless it displays a currently valid OMB control number.

1. Type of submission, new, revision, or extension: Extension.

2. The title of the information collection: Generic Letter 91-02, "Reporting Mishaps Involving LLW Forms Prepared for Disposal."

3. The form number, if applicable: N/A.

4. How often the collection is required: Reports are made only when the licensee or waste processor experiences a mishap that is reportable under the guidelines described in the Generic Letter.

5. Who will be required or asked to report: Nuclear power reactor licensees and Agreement State and non-Agreement State waste processors and disposal site operators.

6. An estimate of the number of responses: 34.

7. The estimated number of annual respondents: 34.

8. An estimate of the number of hours needed annually to complete the requirement or request: 272 hours (an average of 8 hours per response).

9. An indication of whether Section 3507(d), Pub. L. 104-13 applies: Not applicable.

10. Abstract: Generic Letter 91-02 encourages voluntary reporting (by both waste form generators and processors) of information concerning mishaps to low-level radioactive waste (LLW) forms prepared for disposal. The information is used by NRC to determine whether follow-up action is necessary to assure protection of public health and safety.

A copy of the final supporting statement may be viewed free of charge at the NRC Public Document Room, 2120 L Street, NW (lower level), Washington, DC. OMB clearance requests are available at the NRC worldwide web site (<http://www.nrc.gov/NRC/PUBLIC/OMB/index.html>). The document will be available on the NRC home page site for 60 days after the signature date of this notice.

Comments and questions should be directed to the OMB reviewer by November 26, 1999: Erik Godwin, Office of Information and Regulatory Affairs (3150-0156), NEOB-10202, Office of Management and Budget, Washington, DC 20503.

Comments can also be submitted by telephone at (202) 395-3087.

The NRC Clearance Officer is Brenda Jo. Shelton, 301-415-7233.

Dated at Rockville, Maryland, this 20th day of October 1999.

For the Nuclear Regulatory Commission.

Brenda Jo. Shelton,

NRC Clearance Officer, Office of the Chief Information Officer.

[FR Doc. 99-27951 Filed 10-25-99; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket No. 40-8968-ML; ASLBP No. 95-706-01-ML]

Hydro Resources, Inc.; Notice of Reconstitution

Pursuant to the authority contained in 10 CFR 2.721 and 2.1207, the Presiding Officer in the captioned 10 CFR part 2, Subpart L proceeding is hereby replaced by appointing Administrative Judge Thomas S. Moore as Presiding Officer in place of Administrative Judge Peter B. Bloch.

All correspondence, documents, and other material shall be filed with the Presiding Officer in accordance with 10 CFR 2.1203. The address of the new Presiding Officer is: Administrative Judge Thomas S. Moore, Atomic Safety and Licensing Board Panel, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555-0001.

Issued at Rockville, Maryland, this 20th day of October 1999.

G. Paul Bollwerk III,

Chief Administrative Judge, Atomic Safety and Licensing Board Panel.

[FR Doc. 99-27949 Filed 10-25-99; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-315 and 50-316]

Indiana Michigan Power Company; Notice of Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of amendments to Facility Operating License Nos. DPR-58 and DPR-74 issued to Indiana Michigan Power Company (the licensee) for operation of the Donald C. Cook Nuclear Power Plant, Units 1 and 2, located in Berrien County, Michigan.

The proposed amendments involve movement of loads in excess of the design-basis seismic capability of the auxiliary building load handling equipment and structures. The proposed amendment requests approval to move the steam generator sections through the auxiliary building and to disengage crane travel interlocks, and also requests relief from performance of Technical Specification Surveillance Requirement 4.9.7.1.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not: (1) Involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Does the change involve a significant increase in the probability of occurrence or

consequences of an accident previously evaluated?

No. NUREG-0612, "Control of Heavy Loads at Nuclear Power Plants," generically evaluates the probabilities for a heavy load handling event that could result in consequences that exceed 25% of 10 CFR 100 limits. The NRC determined, assuming heavy load handling in accordance with NUREG-0612 guidelines, that the associated risks are acceptable based on the very low likelihood of a load drop. The proposed activity will be performed in accordance with NUREG-0612 as approved for Donald C. Cook Nuclear Plant (CNP) and will be similar to the heavy loads program reviewed, approved, and demonstrated effective during the Unit 2 SGRP (Steam Generator Repair Project). The cranes feature single-failure-proof hoisting and braking systems in accordance with NUREG-0554, "Single-Failure-Proof Cranes for Nuclear Power Plants," and are evaluated to safely retain the load in the unlikely event of the safe shutdown earthquake (SSE). As such, this change does not introduce any new accident precursors or initiators and there is not a significant increase in the probability of previously evaluated accidents.

Administrative controls substitute for crane travel interlocks during the lifts to prevent loads from being carried over spent fuel assemblies. In addition, a load path evaluation has determined that, in the unlikely event of a load drop, requirements for safe shutdown of the operating unit, decay heat removal, and spent fuel pool cooling continue to be satisfied. As a result, there is no significant increase in the consequences of a load drop. Based on the above, the probability of occurrence and the consequences of accidents previously evaluated are not increased.

2. Does the proposed change create the possibility of a new or different kind of accident from any accident previously evaluated?

No. The potential accident involved in the proposed change is a design basis seismic event during load handling. The NUREG-0554 guideline for crane seismic capability is safe retention of the load during an SSE. A current engineering study demonstrates that the SG [steam generator] sections are safely retained by the cranes during load handling even in the unlikely event of an SSE. Although the crane travel interlocks are disengaged during the lifts, administrative controls prevent loads from being carried over the spent fuel pool. Furthermore, the load path, methods, and types of loads are similar to those previously reviewed and approved for the Unit 2 SGRP. That review also found that the possibility of a new or different kind of accident was not created. The current reviews and analyses for the Unit 1 SGRP have not identified a credible new kind of accident or one that is different from the evaluated load drop scenario. Based on the above, the possibility of a new or different kind of accident from any previously evaluated is not created.

3. Does the proposed change involve a significant reduction in a margin of safety?

No. Handling of heavy loads during the proposed activity will be in accordance with the guidelines of NUREG-0612 (including

appropriate codes and standards) as approved for CNP and will be similar to the heavy loads program previously approved for the Unit 2 SGRP. Administrative controls substitute for crane travel interlocks during the lifts to ensure that no loads are carried over spent fuel assemblies. The loads will be lifted by cranes with the single-failure proof features specified by NUREG-0554. For these loads, the design basis seismic capability of the load handling equipment and structures is exceeded. However, the likelihood of a seismic event coincident with the limited lift times for these loads is very remote. Furthermore, an evaluation of these lifts that considers the conservatism inherent in the design basis calculations concludes that the loads are safely retained even in the event of an SSE. Based on the above, the proposed change does not involve a significant reduction in the margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92 are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination.

Normally, the Commission will not issue the amendment until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish in the **Federal Register** a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

Written comments may be submitted by mail to the Chief, Rules and Directives Branch, Division of Administrative Services, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and should cite the publication date and page number of this **Federal Register** notice. Written comments may also be delivered to Room 6D59, Two White Flint North, 11545 Rockville