

NATIONAL TRANSPORTATION SAFETY BOARD

Public Hearing

The National Transportation Safety Board will convene a public hearing beginning at 9 a.m., local time on Monday, September 13, 1999, at the Ambassador West, Wyndham Grand Heritage Hotel, 1300 North State Parkway, Chicago, Illinois concerning the *Investigation of the Collision and Derailment of Amtrak Train No. 59, the City of New Orleans, with an East Bound Tractor Semi-trailer Truck at Railroad/Highway Grade Crossing, near Bourbonnais, Illinois, on March 15, 1999*. For more information, contact James S. Dunn, NTSB Office of Highway Safety at (202) 314-6436 or Terry N. Williams, NTSB Office of Public Affairs at (202) 314-6100.

Dated: September 1, 1999.

Rhonda Underwood,

Federal Register Liaison Officer.

[FR Doc. 99-23232 Filed 9-7-99; 8:45 am]

BILLING CODE 7533-01-M

NATIONAL WOMEN'S BUSINESS COUNCIL

Sunshine Act Notice

AGENCY: National Women's Business Council.

ACTION: Notice of Meeting.

SUMMARY: In accordance with the Women's Business Ownership Act, Public Law 105-135 as amended, the National Women's Business Council (NWBC) announces a forthcoming Council meeting and joint meeting of the NWBC and Interagency Committee on Women's Business Enterprise. The meetings will cover action items worked on by the National Women's Business Council and the Interagency Committee on Women's Business Enterprise included by not limited to procurement, access to capital and training.

DATES: September 23, 1999.

ADDRESSES: Council Meeting & Joint Meeting. The White House/Old Executive Office Building/(17th & Penn. Entrance), Washington, DC. Council Meeting, S-476, 9 a.m. to 10 a.m., Joint Meeting, Indian Treaty Room, 10 a.m. to 12 p.m.

Note: No admittance without prior official clearance. Please have a photo ID.

STATUS: Open to the public.

CONTACT: National Women's Business Council, 409 Third Street, SW., 8th Floor, Washington, DC 20024, (202) 205-3850.

Note: Please call by September 13, 1999.

Gilda Presley,

Administrative Officer,

National Women's Business Council.

[FR Doc. 99-23481 Filed 9-3-99; 3:57 pm]

BILLING CODE 6820-AR-M

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-295 and 50-304]

Commonwealth Edison Company; (Zion Nuclear Power Station, Units 1 and 2); Exemption

I.

Commonwealth Edison Company (ComEd or the licensee) is the holder of Facility Operating License Nos. DPR-39 and DPR-48, which authorize the licensee to possess the Zion Nuclear Power Station (ZNPS). The license states, among other things, that the facility is subject to all the rules, regulations, and orders of the US Nuclear Regulatory Commission (the Commission or NRC) now or hereafter in effect. The facility consists of two pressurized-water reactors located at the ComEd site on the west shore of Lake Michigan about 40 miles north of Chicago, Illinois, in the extreme eastern portion of the city of Zion, Illinois (Lake County). The facility is permanently shut down and defueled, and the licensee is no longer authorized to operate or place fuel in the reactor.

II.

Section 50.12(a) of 10 CFR, "Specific exemption," states that. * * *

The Commission may, upon application by any interested person, or upon its own initiative, grant exemptions from the requirements of the regulations of this part, which are: (1) Authorized by law, will not present an undue risk to the public health and safety, and are consistent with the common defense and security. (2) The Commission will not consider granting an exemption unless special circumstances are present.

Section 50.12(a)(2)(ii) of 10 CFR states that special circumstances are present when "Application of the regulation in the particular circumstances would not serve the underlying purpose of the rule or is not necessary to achieve the underlying purpose of the rule. * * *" The underlying purpose of sections 50.47(b) and 50.47(c)(2) is to ensure that there is reasonable assurance that adequate protective measures can and will be taken in the event of a radiological emergency, to establish

plume exposure and ingestion pathway emergency planning zones for nuclear power plants, and to ensure that licensees maintain effective offsite and onsite emergency plans.

By letter dated April 13, 1999, ComEd requested an exemption from certain provisions of 10 CFR 50.47(b) and 10 CFR 50.47(c)(2) on the basis that the permanently shutdown and defueled condition of the ZNPS had substantially reduced the risk to public health and safety. In addition, the licensee submitted a proposed Defueled Station Emergency Plan (DSEP) for NRC's approval. The DSEP proposed to discontinue offsite emergency planning activities and to reduce the scope of onsite emergency planning. Thus, exemptions from certain provisions of 10 CFR 50.47(b) and 50.47(c)(2) are required to implement the proposed DSEP to maintain compliance with the regulation.

By letter dated April 13, 1999, and supplemental letters dated July 8, July 19, and August 30, 1999, the licensee also submitted an analysis of the radiological consequences of a postulated event, an analysis to determine the maximum Zircaloy cladding temperature in the spent fuel pool (SFP) with the fuel exposed to an air environment, and an analysis to determine the potential upper limit radiation fields at the exclusion area boundary.

III.

The licensee stated that special circumstances exist at ZNPS because of the station's permanently shutdown and defueled condition. The standards in 10 CFR 50.47(b) and the requirements in 10 CFR 50.47(c)(2) were developed taking into consideration the risks associated with operation of a nuclear power reactor at its licensed full-power level. The risks include the potential for an accident with offsite radiological dose consequences. There are no design basis accidents or other credible events for ZNPS that would result in a radiological dose beyond the exclusion area boundary that would exceed the Environmental Protection Agency's (EPA) Protective Action Guidelines (PAGs). Therefore, the application of all of the standards in 10 CFR 50.47(b) and the requirements of 10 CFR 50.47(c)(2) are not necessary to achieve the underlying purpose of those rules.

The licensee analyzed the heatup characteristics of the spent fuel from a beyond design basis event that results in the complete loss of spent fuel pool (SFP) water, when cooling depends on the natural circulation of air through the spent fuel racks. The licensee presented

the results of an analysis showing that as of June 30, 1999, decay heat could not heat the spent fuel cladding above 482 °C in the event all water was drained from the SFP. The staff reviewed the licensee's analysis and found the licensee's value for peak fuel cladding temperature acceptable. On the basis of a staff determination that fuel cladding will remain intact at this temperature, the staff concluded that a complete loss of water from the ZNPS SFP would not result in a release off site that exceeds the early-phase EPA PAGs.

Although a significant release of radioactive material from the spent fuel is no longer possible in the absence of water cooling, a potential exists for radiation exposure to an offsite individual in the event that shielding of the fuel is lost (a beyond-design-basis event). Water and the concrete pool structure serve as radiation shielding on the sides of the pool. However, water alone provides most of the shielding above the spent fuel. A loss of shielding above the fuel could increase the radiation levels off site because of the gamma rays streaming up out of the pool being scattered back to a receptor at the site boundary. The licensee calculated the offsite radiological impact of a postulated complete loss of SFP water and determined that the gamma radiation dose rate at the exclusion area boundary would be 0.00294 rad per hour at an outside air temperature of 21 °C. At this rate, it would take 14 days for the event to exceed the EPA early-phase PAG of 1 rem. The EPA early-phase PAG is defined as the period beginning at the projected or actual initiation of a release and extending a few days later. The PAGs were developed to respond to a mobile airborne plume that could transport and deposit radioactive material over a large area. In contrast, the radiation field formed by scatter from a drained SFP would be stationary rather than moving and would not cause transport or deposition of radioactive materials. The 14 days available for action allow sufficient time to develop and implement mitigative actions and provide confidence that additional offsite measures could be taken without planning if efforts to reestablish shielding over the fuel are delayed.

The standards and requirements that remain in effect are listed in Attachment 1 to the licensee's letter of April 13, 1999, and Attachment 2 to the licensee's letter of July 8, 1999. On the basis of this review, the staff finds that the radiological consequences of accidents possible at ZNPS are substantially lower than those at an operating plant. The upper bound of offsite dose

consequences limits the highest attainable emergency class to the alert level. In addition, because of the reduced consequences of radiological events still possible at the site, the scope of the onsite emergency preparedness organization may be reduced. Thus, the underlying purpose of the regulations will not be adversely affected by eliminating offsite emergency planning activities or reducing the scope of onsite emergency planning. Accordingly, the Commission has determined that special circumstances as defined in 10 CFR 50.12(a)(2)(ii) exist.

IV.

The Commission has determined that, pursuant to 10 CFR 50.12, the exemption is authorized by law, will not present an undue risk to the public health and safety and is consistent with the common defense and security, and is otherwise in the public interest. Therefore, the Commission hereby grants Commonwealth Edison Company an exemption from certain requirements of 10 CFR 50.47(b) and 10 CFR 50.47(c)(2).

Pursuant to 10 CFR 51.32, the Commission has determined that granting of this exemption will have no significant impact on the environment (64 FR 45981).

This exemption is effective upon issuance.

Dated at Rockville, Maryland, this 31st day of August 1999.

For the Nuclear Regulatory Commission.

John A. Zwolinski,

Director, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.

[FR Doc. 99-23297 Filed 9-7-99; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

Tennessee Valley Authority

[Docket No. 50-390]

Notice of Partial Withdrawal of Application for Amendment to Facility Operating License

The Nuclear Regulatory Commission (the Commission) has granted a request by the Tennessee Valley Authority (the Licensee) to withdraw the remainder of its October 23, 1996, application for proposed amendment to Facility Operating License No. NPF-90 for the Watts Bar Nuclear Plant, located in Rhea County, Tennessee.

The remaining portion of the application that was not approved by license amendment number 6, issued on

July 28, 1997, proposed the installation of spent fuel racks in the cask pit area of the spent fuel pool for an additional 225 storage spaces and the use of an impact shield over the fuel in the cask pit when heavy loads are moved near or across the cask pit area.

The Commission had previously issued a Notice of Consideration of Issuance of Amendment published in the **Federal Register** on April 2, 1997 (62 FR 15733). However, by letter dated July 22, 1999, the licensee withdrew that portion of the proposed amendment related to storage in the cask pit.

For further details with respect to this action, see the application for amendment dated October 23, 1996, Amendment to Facility Operating License Number 6 issued on July 28, 1997, and the licensee's letter dated July 22, 1999. The above documents are available for public inspection at the Commission's Public Document room, the Gelman Building, 2120 L Street, NW., Washington, DC and at the local public document room located at the Chattanooga-Hamilton County Library, 1001 Broad Street, Chattanooga, TN 37402.

Dated at Rockville, Maryland, this 1st day of September 1999.

For the Nuclear Regulatory Commission.

Robert E. Martin,

Senior Project Manager, Section 2, Project Directorate II, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.

[FR Doc. 99-23299 Filed 9-7-99; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

Postponement of Public Workshop To Develop a Standard Review Plan for Decommissioning

AGENCY: Nuclear Regulatory Commission.

ACTION: Postponement of public workshop.

SUMMARY: This notice announces the postponement of one of the public workshops the Nuclear Regulatory Commission (NRC) is sponsoring to solicit input from stakeholders during the development of a Standard Review Plan (SRP) and other guidance for decommissioning nuclear facilities.

SUPPLEMENTARY INFORMATION: On October 21, 1998, NRC announced that it was sponsoring a series of public workshops to support the staff's development of an SRP and other guidance for the decommissioning of nuclear facilities. On November 18,