

facility crane. USEC is committing to ANSI B30.2-1990, "Overhead and Gantry Cranes" for the hoist brakes on the cranes.

Basis for finding of no significance:

1. The proposed amendment will not result in a change in the types or significant increase in the amounts of any effluents that may be released offsite.

The proposed changes to revise the design requirements for the cranes in C-310, C-315, and C-360 have no effect on the generation or disposition of effluents. Therefore, the proposed TSR modifications will not result in a change to the types or amount of effluents that may be released offsite.

2. The proposed amendment will not result in a significant increase in individual or cumulative occupational radiation exposure.

The proposed TSR revisions will not change or increase maintenance, testing or operational requirements for the affected equipment; implementation of the revised TSRs will not increase exposure. The changes do not relate to controls used to minimize occupational radiation exposures. Therefore, the changes will not result in a significant increase in individual or cumulative occupational radiation exposure.

3. The proposed amendment will not result in a significant construction impact.

The proposed changes will not result in any building construction, therefore, there will be no construction impacts.

4. The proposed amendment will not result in a significant increase in the potential for, or radiological or chemical consequences from, previously analyzed accidents.

The proposed TSR changes involve a change to the description of the safety features on the cranes in the withdrawal and toll transfer and sampling facilities. The current TSRs specify the type of brakes on the cranes. The proposed TSR would require that the brake designs comply with the requirements of the standard on cranes (ANSI B30.2-1990). The brakes will continue to perform their safety function. The change to the design requirements does not increase the probability of occurrence or consequences of any postulated accident currently identified in the safety analysis report.

5. The proposed amendment will not result in the possibility of a new or different kind of accident.

The proposed TSR modifications will require the brakes to comply with ANSI B30.2-1990. The brakes will continue to perform their safety function. The specific type of brake required will no longer be specified in the TSR. The

proposed changes will not create the possibility of a new or different type of equipment malfunction or a new or different type of accident.

6. The proposed amendment will not result in a significant reduction in any margin of safety.

The proposed TSR changes involve a change to the description of the brake safety feature. Instead of specifying the type of brake, the TSR will commit to a brake design that complies with the requirements of the industry standard for cranes (ANSI B30.2-1990). Although the previous brake designs complied with the standard, it was not required by the TSR. The safety function of the brakes remains unchanged and the brakes will continue to perform their safety function. As such, the changes do not decrease the margin of safety.

7. The proposed amendment will not result in an overall decrease in the effectiveness of the plant's safety, safeguards or security programs.

Implementation of the proposed changes do not change the safety, safeguards, or security programs. Therefore, the effectiveness of the safety, safeguards, and security programs is not decreased.

Effective date: The amendment to Certificate of Compliance GDP-1 becomes effective 30 days after being signed by the Director, Office of Nuclear Material Safety and Safeguards.

Certificate of Compliance No. GDP-1: Amendment will revise TSRs 2.1.5.2 and 2.3.5.2 to change the design requirement for the crane brakes in the C-310, C-315, and C-360 facilities.

Local Public Document Room location: Paducah Public Library, 555 Washington Street, Paducah, Kentucky 42003.

Dated at Rockville, Maryland, this 18th day of November 1998.

For the Nuclear Regulatory Commission.

Elizabeth Q. Ten Eyck,

Acting Director, Office of Nuclear Material Safety and Safeguards.

[FR Doc. 98-31812 Filed 11-27-98; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

[Docket No. 50-390]

Tennessee Valley Authority (Watts Bar Nuclear Plant, Unit 1); Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (the Commission or NRC) is considering issuance of an amendment to Facility Operating

License No. NPF-90, issued to Tennessee Valley Authority (the licensee), for operation of the Watts Bar Nuclear Plant (WBN), Unit 1, located in Rhea County, Tennessee.

Environmental Assessment

Identification of the Proposed Action

The proposed amendment would include provisions in Technical Specification (TS) 4.3 which allows for the storage of fuel assemblies having a maximum nominal enrichment of 5.0 weight percent (w/o) Uranium 235 (U-235) in the new fuel storage racks and would revise requirements governing the placement of fuel assemblies in the new fuel storage pit. The proposed action is in accordance with the licensee's application for amendment dated May 6, 1998, as supplemented on June 5, 1998.

The Need for the Proposed Action

The proposed changes are needed so that the licensee can use higher fuel enrichment to provide the flexibility of extending the fuel irradiation and to permit operation for longer fuel cycles.

Environmental Impacts of the Proposed Action

The Commission has completed its evaluation of the proposed revisions to the TS. The proposed revisions would permit use of fuel assemblies enriched to a maximum nominal of 5.0 w/o U-235. The safety considerations associated with reactor operation with higher enrichment and extended irradiation have been evaluated by the NRC staff. The staff has concluded that such changes would not adversely affect plant safety. The proposed changes have no adverse effect on the probability of any accident. The higher enrichment, with increased fuel burnup, may slightly change the mix of fission products that might be released in the event of a serious accident, but such small changes would not significantly affect the consequences of serious accidents. No changes are being made in the types or amounts of any radiological effluents that may be released offsite. There is no significant increase in the allowable individual or cumulative occupational radiation exposure.

The environmental impacts on the uranium fuel cycle and transportation resulting from the use of higher enrichment fuel and extended irradiation were discussed in the NRC staff Environmental Assessment and Finding of No Significant Impact published in the **Federal Register** on February 29, 1988 (53 FR 6040). These impacts were also discussed in the staff

assessment entitled, "NRC Assessment of the Environmental Effects of Transportation Resulting from Extended Fuel Enrichment and Irradiation," dated July 7, 1988. This assessment was published in connection with an Environmental Assessment related to the Shearon Harris Nuclear Plant, Unit 1, which was published in the **Federal Register** (53 FR 30355) on August 11, 1988, as corrected on August 24, 1988 (53 FR 32322). As indicated therein, the environmental cost contribution of an increase in the fuel enrichment of up to 5.0 w/o percent U-235 and irradiation limits of up to 60,000 gigawatt days per metric ton (GWD/MT) are either unchanged or may, in fact, be reduced from those summarized in 10 CFR 50.51(b), Table S-3, and in Table S-4 as set forth in 10 CFR 51.52(c). These findings are applicable to the proposed increase at Watts Bar given that the proposal involves fuel enrichment of up to 5.0 w/o U-235 and burnup of less than 60,000 GWD/MT. Accordingly, the Commission concludes that there are no significant radiological environmental impacts associated with the proposed amendment.

With regard to potential non-radiological impacts of reactor operation with higher enrichment and extended irradiation, the proposed action involves features located entirely within the restricted area as defined in 10 CFR part 20. It does not affect non-radiological plant effluents and has no other environmental impact. Accordingly, the Commission concludes that there are no significant non-radiological environmental impacts associated with the proposed action.

Alternatives to the Proposed Action

Since the Commission concluded that there are no significant environmental effects that would result from the proposed action, any other alternative would have equal or greater environmental impacts and need not be evaluated.

The principal alternative would be to deny the requested amendment (no-action alternative). This would not reduce the environmental impact of plant operations and would result in reduced operational flexibility.

Alternative Use of Resources

This action does not involve the use of any resources not previously considered in the Final Environmental Statement for WBN, Units 1 and 2, dated April 1995.

Agencies and Persons Consulted

In accordance with its stated policy, on October 22, 1998, the staff consulted

with the Tennessee State official, Mr. E. Nanney of the Division of Radiological Health, regarding the environmental impact of the proposed action. The State official had no comments.

Finding of No Significant Impact

The staff has reviewed the proposed modification to WBN, Unit 1, TS relative to the requirements set forth in 10 CFR Part 51. Based upon the environmental assessment, the staff has concluded that there are no significant radiological or non-radiological impacts associated with the proposed action and that the proposed license amendment will not have a significant effect on the quality of the human environment. Therefore, the Commission has determined, pursuant to 10 CFR 51.31, not to prepare an environmental impact statement for the proposed amendment.

For further details with respect to the proposed action, see the licensee's letter dated May 6, 1998, as supplemented by letter dated June 5, 1998, which are available for public inspection at the Commission's Public Document Room, The Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Chattanooga-Hamilton County Library, 1001 Broad Street, Chattanooga, Tennessee.

Dated at Rockville, Maryland, this 18th day of November 1998.

For the Nuclear Regulatory Commission.

Frederick J. Hebdon,

Director, Project Directorate II-3, Division of Reactor Projects—I/II, Office of Nuclear Reactor Regulation.

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NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards, Subcommittee Meeting on Thermal-Hydraulic Phenomena; Notice of Meeting

The ACRS Subcommittee on Thermal-Hydraulic Phenomena will hold a meeting on December 16-17, 1998, Room T-2B3, 11545 Rockville Pike, Rockville, Maryland.

A portion of this meeting will be closed to public attendance to discuss Westinghouse proprietary information pursuant to 5 U.S.C. 552b(c)(4).

The agenda for the subject meeting shall be as follows:

Wednesday, December 16, 1998—8:30 a.m. until the conclusion of business.

Thursday, December 17, 1998—8:30 a.m. until the conclusion of business.

The Subcommittee will discuss the application of the Westinghouse Electric Company's WCOBRA/TRAC best-estimate large-break LOCA code to nuclear power plants with upper head plenum injection; the NRC Thermal-Hydraulic Code Review Action Plan; and the status of the NRC thermal-hydraulic research program. The purpose of this meeting is to gather information, analyze relevant issues and facts, and to formulate proposed positions and actions, as appropriate, for deliberation by the full Committee.

Oral statements may be presented by members of the public with the concurrence of the Subcommittee Chairman; written statements will be accepted and made available to the Committee. Electronic recordings will be permitted only during those portions of the meeting that are open to the public, and questions may be asked only by members of the Subcommittee, its consultants, and staff. Persons desiring to make oral statements should notify the cognizant ACRS staff engineer named below five days prior to the meeting, if possible, so that appropriate arrangements can be made.

During the initial portion of the meeting, the Subcommittee, along with any of its consultants who may be present, may exchange preliminary views regarding matters to be considered during the balance of the meeting.

The Subcommittee will then hear presentations by and hold discussions with representatives of Westinghouse, the NRC staff, and other interested persons regarding this review.

Further information regarding topics to be discussed, whether the meeting has been canceled or rescheduled, the scheduling of sessions which are open to the public, and the Chairman's ruling on requests for the opportunity to present oral statements and the time allotted therefor, can be obtained by contacting the cognizant ACRS staff engineer, Mr. Paul A. Boehnert (telephone 301/415-8065) between 7:30 a.m. and 4:15 p.m. (EST). Persons planning to attend this meeting are urged to contact the above named individual one or two working days prior to the meeting to be advised of any potential changes to the agenda, etc., that may have occurred.

Dated: November 23, 1998.

Noel F. Dudley,

Acting Chief, Nuclear Reactors Branch.

[FR Doc. 98-31811 Filed 11-27-98; 8:45 am]

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