

Place: Rooms 310, 320, 330, 340, 360, 370, 380, 390, 680, and 1120, National Science Foundation, 4201 Wilson Boulevard, Arlington, VA 22230.

Type of Meeting: Closed.

Contact Person: Dr. Ming Leu, Program Director, Manufacturing, Machines, and Equipment, Dr. Delcie Durham, Program Director, Material Processes and Manufacturing, Dr. George A. Hazelrigg, Program Director, Design and Integration Engineering, Dr. Larry Seiford, Program Director, Operations Research and Production Systems, (703), 306-1330, National Science Foundation, 4201 Wilson Boulevard, Arlington, VA 22230.

Purpose of Meeting: To provide advice and recommendations concerning proposals submitted to the NSF for financial support.

Agenda: To review and evaluate unsolicited proposals as part of the selection process for awards.

Reason for Closing: The proposals being reviewed include information of proprietary or confidential nature, including technical information, financial data such as salaries, and personal information concerning individuals associated with the proposals. These matters that are exempt under 5 U.S.C. 552b(c) (4) and (6) of the Government in the Sunshine Act.

Dated: November 23, 1998.

M. Rebecca Winkler,

Committee Management Officer.

[FR Doc. 98-31725 Filed 11-27-98; 8:45 am]

BILLING CODE 7555-01-M

NATIONAL SCIENCE FOUNDATION

Special Emphasis Panel in Mathematical Sciences; Notice of Meeting

In accordance with the Federal Advisory Committee Act (Pub. L. 92-463, as amended), the National Science Foundation announces the following meeting.

Name and Committee Code: Special Emphasis in Mathematical Sciences (1204).

Date and Time: December 14-16, 1998; 8:30 A.M. until 5:00 P.M.

Place: Rooms 320, 1020, 1060, National Science Foundation, 4201 Wilson Boulevard, Arlington, VA 22230.

Type of Meeting: Closed.

Contact Person: Drs. Keith N. Crank, James Rosenberger, and Javier Rojo, Program Directors, National Science Foundation, 4201 Wilson Boulevard, Arlington, VA 22230. Telephone: (703) 306-1870.

Purpose of Meeting: To provide advice to Program Officers concerning proposals submitted to NSF for financial support.

Agenda: To review and evaluate proposals for the Statistics and Probability Program, as part of the selection process for awards.

Reason for Closing: The proposals being reviewed include information of a proprietary or confidential nature, including technical information; financial data, such as salaries and personal information concerning individuals associated with the proposals.

These matters are exempt under 5 U.S.C. 552b(c) (4) and (6) of the Government in the Sunshine Act.

Dated: November 23, 1998.

M. Rebecca Winkler,

Committee Management Officer.

[FR Doc. 98-31723 Filed 11-27-98; 8:45 am]

BILLING CODE 7555-01-M

NUCLEAR REGULATORY COMMISSION

[Docket 70-7001]

Notice of Amendment to Certificate of Compliance GDP-1 for the U.S. Enrichment Corporation Paducah Gaseous Diffusion Plant Paducah, Kentucky

The Director, Office of Nuclear Material Safety and Safeguards, has made a determination that the following amendment request is not significant in accordance with 10 CFR 76.45. In making that determination, the staff concluded that: (1) There is no change in the types or significant increase in the amounts of any effluents that may be released offsite; (2) there is no significant increase in individual or cumulative occupational radiation exposure; (3) there is no significant construction impact; (4) there is no significant increase in the potential for, or radiological or chemical consequences from, previously analyzed accidents; (5) the proposed changes do not result in the possibility of a new or different kind of accident; (6) there is no significant reduction in any margin of safety; and (7) the proposed changes will not result in an overall decrease in the effectiveness of the plant's safety, safeguards or security programs. The basis for this determination for the amendment request is shown below.

The NRC staff has reviewed the certificate amendment application and concluded that it provides reasonable assurance of adequate safety, safeguards, and security, and compliance with NRC requirements. Therefore, the Director, Office of Nuclear Material Safety and Safeguards, is prepared to issue an amendment to the Certificate of Compliance for the Paducah Gaseous Diffusion Plant. The staff has prepared a Compliance Evaluation Report which provides details of the staff's evaluation.

The NRC staff has determined that this amendment satisfies the criteria for a categorical exclusion in accordance with 10 CFR 51.22(c)(19). Therefore, pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared for this amendment.

USEC or any person whose interest may be affected may file a petition, not exceeding 30 pages, requesting review of the Director's Decision. The petition must be filed with the Commission not later than 15 days after publication of this **Federal Register** Notice. A petition for review of the Director's Decision shall set forth with particularity the interest of the petitioner and how that interest may be affected by the results of the decision. The petition should specifically explain the reasons why review of the Decision should be permitted with particular reference to the following factors: (1) The interest of the petitioner; (2) how that interest may be affected by the Decision, including the reasons why the petitioner should be permitted a review of the Decision; and (3) the petitioner's areas of concern about the activity that is the subject matter of the Decision. Any person described in this paragraph (USEC or any person who filed a petition) may file a response to any petition for review, not to exceed 30 pages, within 10 days after filing of the petition. If no petition is received within the designated 15-day period, the Director will issue the final amendment to the Certificate of Compliance without further delay. If a petition for review is received, the decision on the amendment application will become final in 60 days, unless the Commission grants the petition for review or otherwise acts within 60 days after publication of this **Federal Register** Notice.

A petition for review must be filed with the Secretary, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, Attention: Rulemakings and Adjudications Staff, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW, Washington, DC, by the above date.

For further details with respect to the action see (1) the application for amendment and (2) the Commission's Compliance Evaluation Report. These items are available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW, Washington, DC, and at the Local Public Document Room.

Date of amendment request: April 24, 1998.

Brief description of amendment: The amendment proposes to revise the Technical Safety Requirements (TSRs) which specify the C-310, C-315, and C-360 facilities crane design features credited for safety. The proposed change will specify the design requirement for the crane brakes as opposed to listing the specific type of brake for each

facility crane. USEC is committing to ANSI B30.2-1990, "Overhead and Gantry Cranes" for the hoist brakes on the cranes.

Basis for finding of no significance:

1. The proposed amendment will not result in a change in the types or significant increase in the amounts of any effluents that may be released offsite.

The proposed changes to revise the design requirements for the cranes in C-310, C-315, and C-360 have no effect on the generation or disposition of effluents. Therefore, the proposed TSR modifications will not result in a change to the types or amount of effluents that may be released offsite.

2. The proposed amendment will not result in a significant increase in individual or cumulative occupational radiation exposure.

The proposed TSR revisions will not change or increase maintenance, testing or operational requirements for the affected equipment; implementation of the revised TSRs will not increase exposure. The changes do not relate to controls used to minimize occupational radiation exposures. Therefore, the changes will not result in a significant increase in individual or cumulative occupational radiation exposure.

3. The proposed amendment will not result in a significant construction impact.

The proposed changes will not result in any building construction, therefore, there will be no construction impacts.

4. The proposed amendment will not result in a significant increase in the potential for, or radiological or chemical consequences from, previously analyzed accidents.

The proposed TSR changes involve a change to the description of the safety features on the cranes in the withdrawal and toll transfer and sampling facilities. The current TSRs specify the type of brakes on the cranes. The proposed TSR would require that the brake designs comply with the requirements of the standard on cranes (ANSI B30.2-1990). The brakes will continue to perform their safety function. The change to the design requirements does not increase the probability of occurrence or consequences of any postulated accident currently identified in the safety analysis report.

5. The proposed amendment will not result in the possibility of a new or different kind of accident.

The proposed TSR modifications will require the brakes to comply with ANSI B30.2-1990. The brakes will continue to perform their safety function. The specific type of brake required will no longer be specified in the TSR. The

proposed changes will not create the possibility of a new or different type of equipment malfunction or a new or different type of accident.

6. The proposed amendment will not result in a significant reduction in any margin of safety.

The proposed TSR changes involve a change to the description of the brake safety feature. Instead of specifying the type of brake, the TSR will commit to a brake design that complies with the requirements of the industry standard for cranes (ANSI B30.2-1990). Although the previous brake designs complied with the standard, it was not required by the TSR. The safety function of the brakes remains unchanged and the brakes will continue to perform their safety function. As such, the changes do not decrease the margin of safety.

7. The proposed amendment will not result in an overall decrease in the effectiveness of the plant's safety, safeguards or security programs.

Implementation of the proposed changes do not change the safety, safeguards, or security programs. Therefore, the effectiveness of the safety, safeguards, and security programs is not decreased.

Effective date: The amendment to Certificate of Compliance GDP-1 becomes effective 30 days after being signed by the Director, Office of Nuclear Material Safety and Safeguards.

Certificate of Compliance No. GDP-1: Amendment will revise TSRs 2.1.5.2 and 2.3.5.2 to change the design requirement for the crane brakes in the C-310, C-315, and C-360 facilities.

Local Public Document Room location: Paducah Public Library, 555 Washington Street, Paducah, Kentucky 42003.

Dated at Rockville, Maryland, this 18th day of November 1998.

For the Nuclear Regulatory Commission.

Elizabeth Q. Ten Eyck,

Acting Director, Office of Nuclear Material Safety and Safeguards.

[FR Doc. 98-31812 Filed 11-27-98; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket No. 50-390]

Tennessee Valley Authority (Watts Bar Nuclear Plant, Unit 1); Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (the Commission or NRC) is considering issuance of an amendment to Facility Operating

License No. NPF-90, issued to Tennessee Valley Authority (the licensee), for operation of the Watts Bar Nuclear Plant (WBN), Unit 1, located in Rhea County, Tennessee.

Environmental Assessment

Identification of the Proposed Action

The proposed amendment would include provisions in Technical Specification (TS) 4.3 which allows for the storage of fuel assemblies having a maximum nominal enrichment of 5.0 weight percent (w/o) Uranium 235 (U-235) in the new fuel storage racks and would revise requirements governing the placement of fuel assemblies in the new fuel storage pit. The proposed action is in accordance with the licensee's application for amendment dated May 6, 1998, as supplemented on June 5, 1998.

The Need for the Proposed Action

The proposed changes are needed so that the licensee can use higher fuel enrichment to provide the flexibility of extending the fuel irradiation and to permit operation for longer fuel cycles.

Environmental Impacts of the Proposed Action

The Commission has completed its evaluation of the proposed revisions to the TS. The proposed revisions would permit use of fuel assemblies enriched to a maximum nominal of 5.0 w/o U-235. The safety considerations associated with reactor operation with higher enrichment and extended irradiation have been evaluated by the NRC staff. The staff has concluded that such changes would not adversely affect plant safety. The proposed changes have no adverse effect on the probability of any accident. The higher enrichment, with increased fuel burnup, may slightly change the mix of fission products that might be released in the event of a serious accident, but such small changes would not significantly affect the consequences of serious accidents. No changes are being made in the types or amounts of any radiological effluents that may be released offsite. There is no significant increase in the allowable individual or cumulative occupational radiation exposure.

The environmental impacts on the uranium fuel cycle and transportation resulting from the use of higher enrichment fuel and extended irradiation were discussed in the NRC staff Environmental Assessment and Finding of No Significant Impact published in the **Federal Register** on February 29, 1988 (53 FR 6040). These impacts were also discussed in the staff