

Type of Review: New.

Agency: Employment and Training Administration.

Title: Trade Adjustment Assistance and NAFTA Transitional Adjustment Assistance Program Performance Report.

OMB Number: 1205-New.

Affected Public: State governments.

Total Respondents: 50.

Frequency: Quarterly.

Total Responses: 200.

Average Time per Respondent: 80 hours per quarter.

Estimated Total Burden Hours: 16,000.

Total Burden Cost (capital/startup): \$500,000.

Total Burden Cost (operating/maintaining): \$225,000.

Comments submitted in response to this comment request will be summarized and/or included in the request for Office of Management and Budget approval of the information collection request; they will also become a matter of public record.

Dated: March 4, 1998.

Grant D. Beale,

Acting Director, Office of Trade Adjustment Assistance.

[FR Doc. 98-5913 Filed 3-6-98; 8:45 am]

BILLING CODE 4510-30-M

DEPARTMENT OF LABOR

Office of the Assistant Secretary for Veterans' Employment and Training

Secretary of Labor's Advisory Committee for Veterans' Employment and Training; Notice of Open Meeting

The Secretary's Advisory Committee for Veterans' Employment and Training was established under section 4110 of title 38, United States Code, to bring to the attention of the Secretary, problems and issues relating to veterans' employment and training.

Notice is hereby given that the Secretary of Labor's Advisory Committee for Veterans' Employment and Training will meet on Friday, March 27, 1998, at the U.S. Department of Labor, 200 Constitution Avenue, N.W., Room S-2508, Washington, DC 20210 from 9:00 a.m. to 4:30 p.m.

Written comments are welcome and may be submitted by addressing them to: Ms. Polin Cohanne, Designated Federal Official, Office of the Assistant Secretary for Veterans' Employment and Training, U.S. Department of Labor, 200 Constitution Avenue, N.W., Room S-1315, Washington, D.C. 20210.

The primary items on the agenda are:

- Adoption of Minutes of the Previous Meeting.

- Priority of Services for Veterans in the Employment Service and on American's Job Bank.

- Gulf War Illness.
 - Congressional Report on Status of Legislation Affecting Veterans.
 - Unemployment Insurance Issues.
- The meeting will be open to the public.

Persons with disabilities needing special accommodations should contact Ms. Polin Cohanne at telephone number 202-219-9116 no later than March 18, 1998.

Signed at Washington, D.C. this March 3, 1998.

Espiridion (Al) Borrego,

Assistant Secretary of Labor for Veterans' Employment and Training.

[FR Doc. 98-5912 Filed 3-6-98; 8:45 am]

BILLING CODE 4510-79-M

NATIONAL COMMISSION ON LIBRARIES AND INFORMATION SCIENCE

Sunshine Act Meeting

TIME, DATE, AND PLACE:

Status: Closed

7 April 1998.

9:00-10:30 a.m.—Executive Session to discuss internal personnel matters.

Status: Open

8:00-9:00 a.m.—Linda Hall Library.

10:30-1:15 p.m.—Truman Library, Independence, MO.

1:45-5:00 p.m.—Linda Hall Library, Kansas City, MO.

Status: Open

8 April 1998.

9:00 a.m. to 3:00 p.m.—Linda Hall Library.

MATTERS TO BE DISCUSSED:

Meeting/tour/demonstration, Truman Library.

Tour/demonstration, Linda Hall Library. NCLIS business meeting.

Update on NCLIS projects/plans.

Session with directors of libraries of Big 12+ Library Consortium.

To request further information or to make special arrangements for physically challenged persons, contact Barbara Whiteleather (202-606-9200) no later than one week in advance of the meeting.

Dated: March 3, 1998.

Robert S. Willard,

Acting Executive Director.

[FR Doc. 98-6169 Filed 3-5-98; 3:42 pm]

BILLING CODE 7527-01-M

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-317 and 50-318]

Baltimore Gas and Electric Company; Notice of Withdrawal of Application for Amendment to Facility Operating License

The U.S. Nuclear Regulatory Commission (the Commission) is permitting the withdrawal of Baltimore Gas and Electric Company's (the licensee) application of April 5, 1996, as supplemented November 20, 1996, regarding the proposed amendment to Facility Operating License Nos. DPR-53 and DPR-69 for the Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2, located in Lusby, Maryland.

The proposed amendment would have revised the operating licenses to reflect the new company ownership of Calvert Cliffs Units 1 and 2 and the Independent Spent fuel Storage Installation.

The Commission had previously issued a Notice of Consideration of Issuance of Amendment published in the **Federal Register** on May 22, 1996 (61 FR 25697). However, by letter dated January 30, 1998, the licensee withdrew the proposed change.

For further details with respect to this action, see the application for amendment dated April 5, 1996, and the licensee's letter dated January 30, 1998, which withdrew the application for license amendment. The above documents are available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Calvert County Library, Prince Frederick, Maryland 20678.

Dated at Rockville, Maryland, this 27th day of February 1998.

For the Nuclear Regulatory Commission.

Alexander W. Dromerick,

Senior Project Manager, Project Directorate I-1, Division of Reactor Projects—I/II, Office of Nuclear Reactor Regulation.

[FR Doc. 98-5945 Filed 3-6-98; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-315 and 50-316]

Indiana Michigan Power Company (Donald C. Cook Nuclear Plant, Units 1 and 2); Exemption

I

Indiana Michigan Power Company (the licensee) is the holder of Facility

Operating License Nos. DPR-58 and DPR-74, which authorize operation of the Donald C. Cook Nuclear Plant, Units 1 and 2, respectively. The Donald C. Cook facilities are pressurized-water reactors located at the licensee's site in Berrien County, Michigan. The license provides, among other things, that the facility is subject to all rules, regulations, and orders of the Commission now or hereafter in effect.

II

Section 50.71(e)(4) of Title 10 of the Code of Federal Regulations, "Maintenance of records, making of reports," states, in part, that "Subsequent revisions [to the final safety analysis report (FSAR)] must be filed annually or 6 months after each refueling outage provided the interval between successive updates [to the FSAR] does not exceed 24 months." The two Donald C. Cook facilities share a common FSAR; therefore, this rule requires the licensee to update the same document within 6 months after a refueling outage for either unit.

III

Section 50.12(a), "Specific exemptions," makes the following statement:

The Commission may, upon application by any interested person or upon its own initiative, grant exemptions from the requirements of the regulations of this part, which are—

(1) Authorized by law, will not present an undue risk to the public health and safety, and are consistent with the common defense and security.

(2) The Commission will not consider granting an exemption unless special circumstances are present.

Section 50.12(a)(2)(ii) states that special circumstances are present whenever—

Application of the regulation in the particular circumstances would not serve the underlying purpose of the rule or is not necessary to achieve the underlying purpose of the rule.

IV

As noted in the staff's safety evaluation, the licensee's proposed schedule for FSAR updates will ensure that the FSAR for the Donald C. Cook Nuclear Plant will be kept current within 24 months of the last revision and will not exceed a 24-month maximum interval for submission of updates to the FSAR pursuant to 10 CFR 50.71(e)(4). The Commission has determined that, pursuant to 10 CFR 50.12(a), an exemption is authorized by law, will not present an undue risk to the public health and safety, and is consistent with common defense or security, and is otherwise in the public interest. The Commission has also determined that there are special

circumstances as defined in 10 CFR 50.12(a)(2)(ii) since the recent revision to 10 CFR 50.71(e), intended to decrease the burden associated with submittal of revisions to the FSAR, did not address multiple-unit sites with a common FSAR and provides that FSAR updates must be filed every 24 months. The licensee's proposed exemption provides the decrease in burden which was intended by the revision and, therefore, achieves the underlying purpose of the rule. The Commission hereby grants the licensee an exemption from the requirement of 10 CFR 50.71(e)(4) to submit updates to the FSAR for the Donald C. Cook Nuclear Plant within 6 months of each outage. The licensee will be required to submit updates to the Donald C. Cook Nuclear Plant FSAR once every Unit 1 fuel cycle, but not to exceed 24 months from the last submittal.

Pursuant to 10 CFR 51.32, the Commission has determined that the granting of this exemption will have no significant impact on the quality of the human environment (62 FR 59753).

This exemption is effective upon issuance.

Dated at Rockville, Maryland, this 3d day of March 1998.

For the Nuclear Regulatory Commission.

Samuel J. Collins,

Director, Office of Nuclear Reactor Regulation.

[FR Doc. 98-5947 Filed 3-6-98; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket No. 50-286]

Power Authority of the State of New York; Notice of Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. DPR-64 issued to New York Power Authority for operation of the Indian Point Nuclear Generating Unit No. 3 (IP3) located in Westchester County, New York.

The proposed amendment would change the pressure-temperature and overpressure limits.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended

(the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

(1) Does the proposed license amendment involve a significant increase in the probability or consequences of an accident previously analyzed?

Response:

The proposed license amendment does not involve a significant increase in the probability or consequences of a previously analyzed accident. The pressure-temperature limit changes proposed by this amendment are based on supporting data and evaluation methodologies previously submitted to the NRC in References 2, 3 and 4 [see application dated February 27, 1998]. These limits are based upon the irradiation damage prediction methods of Regulatory Guide 1.99, Revision 2. The LTOPS [low-temperature overpressure protection] changes contained in this submittal have been conservatively adjusted in accordance with the new pressure-temperature limits, in accordance with the information contained in References 2 and 5 [see application dated February 27, 1998] and ASME Code Case N-514.

The revised version of Section 3.1.A.8 clarifies existing requirements related to the OPS [overpressure protection system] system and adds an eight hour completion time for compensating actions, consistent with the STS. The changes to Section 3.1.A.1.h, l, and j revise the requirements associated with the start of an RCP [reactor coolant pump]. These changes improve specification clarity and do not increase the probability or consequences of an accident.

The Technical Specification changes associated with the restriction on SI [safety injection] pumps provides added conservatism to the Technical Specifications and limits the likelihood of an RHR [residual heat removal] overpressurization event. Current plant procedures prohibit actuation of any SI pumps when RHR is in service, except during testing, loss of RHR cooling, or reduced inventory operations. Therefore, the change to the Technical Specifications will not alter current plant operation.

(2) Does the proposed license amendment create the possibility of a new or different kind of accident from any accident previously evaluated?

Response: