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General Wage Determination Publication

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Signed at Washington, D.C. this 9th day of February 1996.

Philip J. Gloss,

Chief, Branch of Construction Wage Determinations.

[FR Doc. 96-3337 Filed 2-15-96; 8:45 am]

BILLING CODE 4510-27-M

NATIONAL TRANSPORTATION SAFETY BOARD

Integrated Bridge Systems: A Public Forum

A public forum on integrated bridge systems—latest in ship control technology, design standards, and human interaction—sponsored by the National Transportation Safety Board will be held March 6-7, 1996, at the Sheraton Premiere (Tysons Corner) Hotel, 8661 Leesburg Pike, Vienna, Virginia. For more information, contact the Marine Division, telephone (202) 382-6860/6863, fax (202) 382-0692 or Pat Cariseo, Office of Public Affairs, Washington, D.C. 20594, telephone (202) 382-0660.

Dated: February 12, 1996.

Bea Hardesty,

Federal Register Liaison Officer.

[FR Doc. 96-3523 Filed 2-15-96; 8:45 am]

BILLING CODE 7533-01-M

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-295 and 50-304]

Commonwealth Edison Company; Zion Nuclear Power Station, Unit Nos. 1 and 2; Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License Nos. DPR-39 and DPR-48, issued to Commonwealth Edison Company (the licensee), for operation of Zion Nuclear Power Station, Unit Nos. 1 and 2, located in Lake County, Illinois.

Environmental Assessment

Identification of the Proposed Action

The proposed amendment will revise the existing Technical Specifications (TS) in its entirety and incorporate the guidance provided in NUREG-1431 (September 1992), "Standard Technical Specifications, Westinghouse Plants." The proposed action is in accordance with the licensee's amendment request dated November 3, 1995, as supplemented on November 22, 1995.

The Need for the Proposed Action

It has been recognized that nuclear safety in all plants would benefit from improvement and standardization of TS. The "NRC Interim Policy Statement on Technical Specification Improvements for Nuclear Power Reactors," (52 FR 3788) contained proposed criteria for defining the scope of technical

specifications. Later, the "NRC Final Policy Statement on TS Improvement for Nuclear Power Reactors," (58 FR 39132) incorporated lessons learned since publication of the interim policy statement and formed the basis for recent revisions to 10 CFR 50.36. The "Final Rule" (60 FR 36953) codified criteria for determining the content of technical specifications. To facilitate the development of standard TS, each reactor vendor owners' group (OG) and the NRC staff developed standard TS. For Westinghouse plants, the Standard Technical Specifications (STS) are NUREG-1431 (September 1992), "Standard Technical Specifications, Westinghouse Plants." This document formed the basis for the Zion Improved Standard TS (ISTS) conversion. The NRC Committee to Review Generic Requirements (CRGR) reviewed the STS, made note of its safety merits, and indicated its support of conversion by operating plants to the STS.

Description of the Proposed Change

The proposed revision to the TS is based on NUREG-1431 and on guidance provided in the Final Policy Statement. Its objective is to completely rewrite, reformat, and streamline the existing TS. Emphasis is placed on human factors principles to improve clarity and understanding. The Bases section has been significantly expanded to clarify and better explain the purpose and foundation of each specification. In addition to NUREG-1431, portions of the existing TS were also used as the basis for the development of the Zion ISTS. Plant specific issues (unique design features, requirements, and operating practices) were discussed at length with the licensee and generic matters with Westinghouse and other OGs.

The proposed changes from the existing TS can be grouped into four general categories. These groupings are characterized as relocated requirements, administrative changes, less restrictive changes involving deletion of requirements, and more restrictive changes; and are as follows:

1. Relocated requirements are items which are in the existing Zion TS, but do not meet the criteria set forth in the Final Policy Statement. The Final Policy Statement establishes a specific set of objective criteria for determining which regulatory requirements and operating restrictions should be included in TS. Relocation of requirements to documents with an established control program allows the TS to be reserved only for those conditions or limitations upon reactor operation which are necessary to obviate the possibility of an

abnormal situation or event giving rise to an immediate threat to the public health and safety, thereby focusing the scope of the TS. In general, the proposed relocation of items from the Zion TS to the Updated Final Safety Analysis Report (UFSAR), appropriate plant specific programs, procedures and ISTS Bases follows the guidance of NUREG-1431. Once these items have been relocated to other licensee controlled documents, the licensee may revise them under the provisions of 10 CFR 50.59 or other NRC staff approved control mechanisms which provide appropriate procedural means to control changes.

2. Administrative changes involve the reformatting and rewording of requirements, consistent with the style of the Westinghouse STS in NUREG-1431, to make the TS more readily understandable to plant operators and other users. These changes are purely editorial in nature or involve the movement or reformatting of requirements without affecting technical content. Application of a standardized format and style will also help ensure consistency is achieved among specifications. During this reformatting and rewording process, no technical changes (either actual or interpretational) to the TS were made unless they were identified and justified.

3. Less restrictive changes and the deletion of requirements involves portions of the existing specifications which provide information that is descriptive in nature regarding the equipment, systems, actions or surveillances, provide little or no safety benefit, and place an unnecessary burden on the licensee. This information is proposed to be deleted from the specifications and, in some instances, moved to the proposed Bases, Updated Final Safety Analysis Report (UFSAR), or procedures. The removal of descriptive information to the Bases of the TS, UFSAR or procedures is permissible, because the Bases, UFSAR or procedures will be controlled through a process which utilizes 10 CFR 50.59. The relaxations of requirements were the result of generic NRC action or other analyses. They have been justified on a case-by-case basis for Zion Nuclear Power Station as described in the Safety Evaluation to be issued with the license amendments.

4. More restrictive requirements are proposed to be implemented in some areas to impose more stringent requirements than presently exist. These more restrictive requirements are being imposed to be consistent with the Westinghouse STS. Such changes have

been made after ensuring the previously evaluated safety analysis was not affected. Also, other more restrictive technical changes have been made to achieve consistency, correct discrepancies, and remove ambiguities from the specification. Examples of more restrictive requirements include: placing a Limiting Condition for Operation (LCO) on plant equipment which is not required by the present TS to be operable; more restrictive requirements to restore inoperable equipment; and more restrictive surveillance requirements.

Environmental Impacts of the Proposed Action

The Commission has completed its evaluation of the proposed revision to the TS. Changes which are administrative in nature have been found to have no effect on the technical content of the TS and are acceptable. The increased clarity and understanding these changes bring to the TS are expected to improve the operators' control of the plant in normal and accident conditions.

Relocation of requirements to other licensee controlled documents does not change the requirements themselves. Future changes to these requirements may be made by the licensee under 10 CFR 50.59 or other NRC approved control mechanisms, which ensures continued maintenance of adequate requirements. All such relocations have been found to be in conformance with the guidelines of NUREG-1431 and the Final Policy Statement, and are, therefore, acceptable.

Changes involving more restrictive requirements have been found to enhance plant safety and to be acceptable.

Changes involving less restrictive requirements have been reviewed individually. When requirements have been shown to provide little or no safety benefit or to place unnecessary burden on the licensee, their removal from the TS was justified. In most cases, relaxations previously granted to individual plants on a plant specific basis were the result of a generic action, or of agreements reached during discussions with the OG and found to be acceptable for Zion. Generic relaxations contained in NUREG-1431 have also been reviewed by the NRC staff and have been found to be acceptable.

In summary, the proposed revisions to the TS were found to provide control of plant operations such that reasonable assurance will be provided that the health and safety of the public will be adequately protected.

These TS changes will not increase the probability or consequences of accidents, no changes are being made in the types of any effluent that may be released off site, and there is no significant increase in the allowable individual or cumulative occupational radiation exposure. Therefore, the Commission concludes that there are no significant radiological environmental impacts associated with the proposed TS amendments.

With regard to potential nonradiological impacts, the proposed amendments involve features located entirely within the restricted area as defined in 10 CFR Part 20. They do not affect nonradiological plant effluents and have no other environmental impact. Therefore, the Commission concludes that there are no significant nonradiological impacts associated with the proposed amendments.

Alternatives to the Proposed Action

Since the Commission has concluded there is no measurable environmental impact associated with the proposed amendments, any alternatives with equal or greater environmental impact need not be evaluated. The principal alternative to this action would be to deny the amendment request. Such action would not reduce the environmental impacts of plant operations.

Alternative Use of Resources

This action does not involve the use of resources not previously considered in connection with the Nuclear Regulatory Commission's Final Environmental Statement dated December 1972, related to the operation of the Zion Nuclear Power Station, Units 1 and 2.

Agencies and Persons Consulted

In accordance with its stated policy, on January 5, 1996, the NRC staff consulted with the Illinois State Official, Mr. Frank Niziolek, Head, Reactor Safety Section, Division of Engineering, Illinois Department of Nuclear Safety; regarding the environmental impact of the proposed action. The State official had no comments.

Finding of No Significant Impact

Based upon the foregoing environmental assessment, the Commission concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the Commission has determined not to prepare an environmental impact

statement for the proposed amendments.

For further details with respect to this action, see the licensee's letter dated November 3, 1995, as supplemented on November 22, 1995, which are available for public inspection at the Commission's Public Document Room, The Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Waukegan Public Library, 128 N. County Street, Waukegan, Illinois 60085.

Dated at Rockville, Maryland, this 9th day of February 1996.

For the Nuclear Regulatory Commission.
Clyde Y. Shiraki,
*Project Manager, Project Directorate III-2,
Division of Reactor Projects—III/IV, Office of
Nuclear Reactor Regulation.*

[FR Doc. 96-3550 Filed 2-15-96; 8:45 am]

BILLING CODE 7590-01-P

[Docket No. 40-8964]

Rio Algom Mining Corp.; Final Finding of No Significant Impact Notice of Opportunity for Hearing

AGENCY: Nuclear Regulatory Commission.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) proposes to amend NRC Source Material License SUA-1548 to allow the licensee, Rio Algom Mining Corp. (Rio Algom), to employ deep well disposal of process waste waters at its Smith Ranch in-situ leach facility as an alternate disposal option for these wastes. An Environmental Assessment was performed by the NRC staff in accordance with the requirements of 10 CFR Part 51. The conclusion of the Environmental Assessment is a Finding of No Significant Impact (FONSI) for the proposed licensing action.

FOR FURTHER INFORMATION CONTACT: Mr. James R. Park, Uranium Recovery Branch, Mail Stop TWFN 7-J9, Division of Waste Management, Office of Nuclear Material Safety and Safeguards, U.S. Nuclear Regulatory Commission, Washington, DC 20555. Telephone 301/415-6699.

SUPPLEMENTARY INFORMATION:

Background

On March 12, 1992, the NRC issued to Rio Algom an NRC Source Material License, SUA-1548, for commercial-scale uranium recovery operations at Rio Algom's Smith Ranch in-situ leach (ISL) facility in Converse County, Wyoming.

The NRC review of Rio Algom's license application is documented in an

Environmental Assessment (EA), issued on January 10, 1992.

Since the issuance of SUA-1548, Rio Algom has deferred commercial-scale operations at the Smith Ranch facility due to the depressed market for uranium. However, Rio Algom has indicated that it plans to commence such operations in the Fall of 1997.

Summary of the Environmental Assessment

Identification of the Proposed Action

The proposed action is an amendment to SUA-1548 to allow Rio Algom to employ deep well injection as an alternate disposal option for process waste waters to be generated at its Smith Ranch ISL facility. The NRC staff's review was conducted in accordance with the requirements of 10 CFR 40.32 and 10 CFR 40.45.

Need for the Proposed Action

Rio Algom requested NRC approval of the proposed action to allow it to employ deep well injection in the disposal of process solutions from various waste streams that would be produced at the Smith Ranch facility. Currently, Rio Algom is required by NRC license to return all liquid effluents from commercial operations to the uranium recovery process circuit or to discharge them to solution evaporation ponds.

Environmental Impacts of the Proposed Action

By this proposed action, Rio Algom is seeking to employ deep well injection to dispose of a variety of process waste streams. These process wastes would be injected at an average of 150 gallons per minute from an injection well drilled to a total depth of 10,100 feet below surface. The wastes would be injected into permeable portions of the Parkman, Teapot, and Teckla formations, at depths below surface ranging from 8700 to 9600 feet. Due to high levels of total dissolved solids, groundwater in these formations has been designated by the State of Wyoming, Department of Environmental Quality (WDEQ), as Class VI water (unsuitable for use).

The Smith Ranch facility is not currently operating. As a result, Rio Algom provided anticipated ranges in concentration of the principal chemical species to be contained in the composite solution. The ranges in concentration of constituents identified by Rio Algom are comparable to concentrations allowed by the NRC at other ISL facilities employing deep well disposal of process fluids.

The NRC staff limited its analysis to a review of the radiological aspects of